Appendix B

Feasibility Study for Conversion of Prairie Island to Natural Gas Fired Generation

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Abstract

This report documents a general feasibility study that examines the conversion of the Prairie Island site from nuclear to natural gas generation. A number of plausible alternatives were investigated. These alternatives involve the replacement or repowering of nuclear capacity with natural gas combustion turbine platforms.

Although all of the scenarios involve some use of existing plant and equipment, the repowering option uses the most existing plant and equipment and in particular employs the existing steam turbine generators. The generation alternatives investigated include simple cycle capacity replacement, combined cycle capacity replacement, and combined cycle repowering. These alternatives are detailed below.

- Replace the nuclear capacity with gas turbine generators running in simple cycle mode
- 2. Replace the nuclear capacity with two standard natural gas combined cycle plants
- Repower one nuclear unit with steam from a combined cycle plant and retire the other nuclear unit
- 4 Repower both nuclear units with steam from two separate combined cycle plants

Budgetary capital and Operation and Maintenance (O&M) cost estimates for each generation scenario are provided. The study provides brief discussions of significant technical and licensing issues that introduce project risk and influence feasibility. The study also includes discussions of key advantages and disadvantages of the various generation alternatives. For each alternative, a complementary real-life example is presented to show a known commercial implementation of a similar project. Supporting data is provided in the appendices.

For reasons identified herein, the combined cycle replacement option (2) and the repower one nuclear unit option (3) provide the most effective alternatives to replace the Prairie Island generating capacity. Accordingly, more detailed information regarding the implementation, construction, and scheduling of these particular alternatives is provided. Option (4) is not a practical engineering solution and is not treated in detail beyond the necessary discussion of the constraints that restrict feasibility. Although the simple cycle option (1) is not nearly as favorable a replacement for the Prairie Island capacity as options (2) and (3), plant cost and other relevant data for simple cycle are provided at certain points for comparison purposes.

Table 1 below, Summary of Prairie Island Natural Gas Generation Alternatives, shows the salient results of this analysis.

Table 1 Summary of Prairie Island Natural Gas Generation Alternatives

Generation Alternative	Net Plant Output (MW)	Unit Net Heat Rate at ISO Conditions (BTU/kwh LHV)	Total Capital Requirement (\$1000)	Normalized Capital Cost (\$/kw)
Simple Cycle Replacement of Both Nuclear Units	999	10539	571,645	572
Combined Cycle Replacement of Both Nuclear Units	1036	6366	643,812	597
3) Repower One Nuclear Unit (4x1)	943	6815	510,921*	542*
* Duct Burners Included	1063*	7298*		
4) Repower Both Nuclear Units (4x1)	1886	6815	NA	NA
5) Present Plant - Nuclear Units	1070	10470 (9783 design)		

Analysis Approach and Key Assumptions

The feasibility study employed EPRI's State of the Art Power Plant (SOAPP) CT workstation to develop the plant financial models. For the repowering case, the GE Gate Cycle workstation was used to determine a plant heat balance and a viable conceptual design. The following list shows significant assumptions and inputs used in the analysis.

- Plant heat rate results are given at the performance point using natural gas as the primary fuel.
- Natural gas supply costs, project development and management costs, and other soft costs such as interest during construction that add to capital cost are included in addition to process capital costs.
- Environmental externalities have not been quantified or monetized.
- The results are presented in 2002 dollars
- Existing equipment not used in the scenarios was assumed to be abandoned-inplace, decommissioning costs were assumed to be unaffected, and demolition costs are excluded.
- Offsite transmission costs such as those that may be needed to preserve system stability are not included. These costs may have a material effect and should be investigated further if a more detailed study is contemplated. A brief discussion of transmission issues is included herein.
- Because this study concerns general feasibility, the plant configurations have not been economically optimized. The costs presented herein reflect approximate costs associated with reasonable and viable plant designs.
- The physical characteristics of the site are deemed adequate for the scenarios.
 Additional site restrictions such as underground obstacles, barriers to construction, or contaminated soils are not contemplated.
- Environmental costs to support BACT controls for NOx are included.
- The repowering analysis is limited to replacing the reactor steam with that from a natural gas CT/HRSG combination. Other forms of repowering such as coal boiler or gasification are not considered herein.
- Existing plant equipment reused in the natural gas generation scenarios is assumed to be in good working order.

Simple Cycle Capacity Replacement

Scenario

The simple cycle capacity replacement scenario involves installation of twelve combustion gas turbines at the PI site operating in simple cycle mode to replace the nuclear capacity.

Description

A simple cycle plant consists of a combustion gas turbine operating in open cycle mode. A simple cycle plant is run intermittently and is principally used for peak shaving. The plant heat rates are less efficient than combined cycle plants, but the plant response time to serve load is faster. Typical startup times are on the order of 20 minutes. Because of their higher heat rates and associated higher variable operating costs, these plants are higher up in the dispatch order and would not be expected to operate more than 15% of the time. A total of 12 units are assumed, with each 6-unit block producing approximately 500 MW. Turbine inlet air fogging was assumed as a performance enhancement. A General Electric 7EA combustion gas turbine with Dry Low Nitrogen (DLN) combustors was chosen as the base unit for this study. The 7EA machine is a typical base unit for large peaking plants. Great River Energy has a six unit peaking plant (Lakefield Junction) in Trimont, MN, which is based on the 7EA platform. The 7EA is also the platform used at Duke's Vermillion Plant in Lincoln County, NC. At 1200 MW, this 16-unit plant is the largest peaking plant in the United States.

Major Retained Equipment and Facilities

For this scenario, the following existing equipment was assumed to be available and incorporated into the cost model: Switchyard and Administration Buildings.

Key Advantages

- The large turbine order (12 units) may allow for some savings on price. Turbine availability concerns have been obviated by recent plant cancellations and reduced order flow to suppliers.
- A simple cycle is an uncomplicated and modular design with the fastest construction schedule, which allows for quick asset mobilization.
- Can be installed with relatively little disruption to the operation of nuclear units

Key Disadvantages

The simple cycle peaking capacity does not replace the baseload capacity lost
with the nuclear unit shutdown. The ability to control system voltage and
frequency within the transmission system may be adversely affected. This may
degrade transmission system reliability. See Transmission Issues section below.

Combined Cycle Capacity Replacement

Scenario

The combined cycle capacity replacement scenario involves the installation of two standard 2x1 natural gas combined cycle plants, each with new steam turbine generators, to replace the nuclear capacity at Prairie Island

Description

A typical combined cycle plant consists of a combustion gas turbine (CTG), matched with an unfired Heat Recovery Steam Generator (HRSG), providing steam to a steam turbine generator (STG). For this analysis, the industry standard 2x1 plant configuration was assumed. That is, two CTGs, each with a matched HRSG, providing steam to a single steam turbine generator was assumed for the base plant. In a combined cycle plant, the gas turbine generators contribute approximately two-thirds of the total plant power. A typical output for this configuration is 500 MW per plant. In order to fully replace the PI generation capacity and utilize the existing transmission capacity, two standard plants are needed.

Combined cycle plants are highly efficient units that are suitable for base load and midrange dispatch. Net thermal efficiencies for these plants are on the order of 53% LHV. The plant is assumed to operate in baseload mode, although it is well suited for cycling duty of approximately 16 hours a day. Combined cycle plants are usually shutdown during weekends and evenings when the spark spread for non-peak power makes these units unprofitable.

The gas turbine platform for this analysis is the Seimens -Westinghouse 501 FD. For these analyses, the gas turbines are assumed to be equipped with Dry Low NOx (DLN) combustors, and each HRSG has an integral Selective Catalytic Reduction (SCR) unit to reduce stack gas NOx emissions.

The Sacramento Municipal Utility District (SMUD) is currently engaged in the design and licensing of a natural gas combined cycle plant at the decommissioned Rancho Seco nuclear power plant facility. This project is known as the Cosumnes Power Plant Project - CPP. According to their submittals to the California Energy Commission (Docket 01-AFC-19), a total of 1000MW of combined cycle replacement power is planned for this project. The proposed plant uses the existing switchyard and some other facilities. The plants are scheduled for construction in two phases consisting of 500 MW each. The first phase is scheduled for commercial operation in 2005 and the second phase, if completed, is scheduled for 2008.

Florida Power and Light (FPL) is currently engaged in the early stages of the siting process for a stand alone combined cycle 550 MW plant to be located adjacent to Exelon Nuclear's Limerick Generating Station. This project is an example of constructing a

natural gas plant at an operating nuclear generation site. Although limited public information has been provided, it appears that there are no plans to shutdown the nuclear units as part of this project or to share any significant equipment. As of June 2002, the NRC was preparing to review the impacts on nuclear operations with input from Exelon, which is a requirement of the Limerick operating license. The siting process has, however, been halted as the township's decision to allow the plant construction has recently been overturned. The following is an excerpt of an article that appeared in the October 3, 2002 edition of the Philadelphia Inquirer.

A three-judge panel in Montgomery County Court on Tuesday overturned an unpopular decision by township officials to allow the plant to be built in the Linfield section. The movement against the gas-powered plant, which opponents argued did not belong in a light-industrial zone, also helped topple the political careers of four township supervisors who backed it. The \$300 million plant was slated to be running at a site near Peco's nuclear power plant by next summer. It would have employed 20 to 25 full-time workers and contributed about \$3 million a year to the tax rolls of Limerick Township, Montgomery County, and the Spring-Ford Area School District. FPL Energy and its local subsidiary, Limerick Partners L.L.C., could not be reached for comment. They have 30 days to appeal the decision to Commonwealth Court.

Major Retained Equipment and Facilities

For this scenario, the following existing equipment was assumed to be available and incorporated into the cost model: Water Treatment System, Switchyard, Circulating Water System, Cooling Tower, Administration Buildings.

Key Advantages

- · High thermal plant efficiencies
- · Relatively short starting times for a baseload unit
- Excellent part-load operating performance and flexible duty cycle
- Standardized design and construction
- Modular design and construction reduces AFUDC
- Fewer design compromises needed to match new equipment with older existing equipment
- Gas turbines can be installed in simple cycle mode prior to full combined cycle mode to reduce the impact of the lost capacity

Key Disadvantages

Higher initial capital costs

Repowering

Discussion

The attractiveness of repowering is usually due to savings from the use of existing equipment permits and public acceptance of the existing site as a generating facility. Repowering projects avoid the cost and uncertainty of siting a new facility while the plant heat rate is typically improved over the existing unit and the capacity of the existing plant increases. In the case of replacing existing fossil-fueled boilers, repowering also can significantly reduce plant emissions. Most repowering projects in the United States have involved replacing a fossil-fueled heat source.

The performance improvements coupled with the reduction in emissions make repowering an efficient choice where capacity additions are needed. A typical increase in repowered output (MW) is triple the original plant output. The concept of repowering involves replacing the original steam generation source with more efficient equipment that is thermally matched to the existing steam turbine generator. A repowering option retains as much auxiliary equipment as possible. Repowering is designed to improve the overall thermal efficiency of the plant while keeping site development costs low and while keeping capital costs low by using existing equipment. Because nuclear fuel costs are much lower than fossil fuels improving the plant heat rate is less of an economic incentive for repowering at Prairie Island

Because of the optimization engineered into the greenfield combined cycle design equipped with integral steam turbine generators, a repowered plant will not be as thermally efficient as a new combined cycle plant. In order for a repowering project to be an efficient use of capital compared to a greenfield generation alternative, the equipment cost savings derived from repowering need to exceed the inherent efficiency advantages of the greenfield alternative for a given amount of deployable MW to the grid. That is, the efficiency difference should not be so great as to result in a material shifting of the dispatch order of the repowered plant over a greenfield alternative. In deregulated markets, an investment in repowering option is not typically warranted if the end result is to simply displace an existing unit in the dispatch order.

Repowering of steam power plants with gas turbine generators and HRSGs is being accomplished in various applications. Colorado Public Service repowered the existing steam turbines at the previously decommissioned Fort St. Vrain nuclear facility in 1999. This plant was originally rated at 330 MW and has been repowered to approximately 720 MW with the installation of three GE 7FA gas turbines and three HRSGs. While there is considerable experience with repowering to replace fossil fueled boilers with gas turbine exhaust (dating to approximately 1960), there have been no nuclear repowering projects other than Fort St. Vrain in the United States.

Florida Power & Light (FP&L) is repowering the 540 MW oil-fired Fort Myers plant with combined cycle technology to ultimately increase plant capacity to approximately 1440 MW. This project provides an example of repowering a steam turbine generator

that is very similar in capacity to the existing Prairie Island steam turbines. Thermal efficiency is expected to increase from approximately 39.6% to 53.7% LHV at ISO load conditions. Six GE Frame 7FA combustion gas turbines and six Foster Wheeler HRSGs with triple pressure and reheat are being installed to replace the oil-fired boiler. The six gas turbines were initially installed in a simple cycle configuration and provided an additional 912 MW from the Fort Myers site. Full combined cycle repowered operation is scheduled for fall of 2002. The cost of this single-unit repowering project was approximately \$450 to 500 million.

Scenarios

The PI repowering scenarios involve installation of combustion turbine generators running in combined cycle using the existing steam turbine generators. The design parameters for the existing steam turbine generators were used in the model. Two scenarios were examined: 1) repower a single unit and, 2) repower both units.

Description

The GE Frame 7FA unit with Dry Low Nitrogen (DLN) combustors was used as the base CTG in the simulation because it provides sufficiently high gas exhaust temperature for the reheat cycle. The efficiency and output of a steam turbine is a function of the gas turbine exhaust temperature. The 7FA is the most widely used unit in modern combined cycle applications. It has an extensive operating history and proven reliability. Siemens-Westinghouse has installed a G class machine with slightly higher efficiencies at a few locations, but these machines do not yet have a detailed history of reliability.

According to the heat balance model, six gas turbines are needed to efficiently repower an existing steam turbine at Prairie Island. The performance of one repowered plant in a 6x1 configuration is estimated as follows.

Net Plant Output - 1418.2 MW

Net Plant Heat Rate - 6599 Btu/kWh LHV

Repowered ST Generator Output - 446.6 MWW (of 535 MW available)

To efficiently operate the existing STGs, six CTGs are needed to replace the steam flow formerly provided by the nuclear reactor. Repowering one nuclear STG results with a more efficient 6x1 configuration results in a site output of approximately 1412 MW, which is approximately 352 MW above current output. Repowering both plants in a 6x1 configuration would result in a site output of 2836 MW, which is 1700 MW above current output. Since these results exceed equipment limits, the 6x1 configuration was not further analyzed. See Transmission Issues section below.

Four CTGs in a 4x1 configuration were used so that current site capacity was matched and output was within known switchyard equipment and transmission limits. Repowering a single nuclear STG with a 4x1 configuration would result in a site output of 943 MW and 1060 MW with a duct burner performance enhancement. Since a duct

burner equipped configuration matches the current nuclear output well, it was used in as the base repowering scenario. Repowering both plants in a 4x1 configuration results in a site output of 1886 MW, which is 826 MW above current output and above known equipment and transmission limits. A 4x1 configuration also allows for future conversion to a 6x1 configuration with an increase from 943 MW to 1412 MW if dictated by system load.

Engineering Issues and the Heat Balance Model

A heat recovery steam generator is most efficient when steam is generated at multiple pressure levels. This contradicts the conventional boiler method of using steam turbine extractions to heat the feedwater. Instead, steam is introduced into the steam turbine at different points, with the steam turbine designed to handle the additional flow at lower pressures. Given the above, the use of the existing feedwater heaters at Prairie Island would rob the HRSGs of heat absorption capability, so the feedwater heaters have been removed from the conceptual design. Since the PI turbines were designed to operate in a saturated steam nuclear cycle, the blades have moisture separation features. The ability to drain the separated moisture has been retained in the model, assuming that the extraction points would be converted into level controlled drip legs of sufficient size to handle the drain capacity.

When a conventional plant steam turbine is repowered with combined cycle steam, the turbine is typically restricted to a maximum amount of exhaust flow. The result of the Low Pressure (LP) steam flow limitation is that the bowl pressure after the throttling valves drops to the point that continuing to use the steam chest costs performance. Most combined cycle steam turbines are designed without a control stage and operate with valves wide open to accommodate rapid fluctuations in heat input to the HRSG, due to a number of variables affecting the gas turbine performance. Load is controlled by changing the load point of the gas turbines. In order to allow the HRSGs to dampen thermal changes, the control stage and steam chest can be removed, but for the purpose of the model, the steam turbine is assumed to operate with the steam chest valves wide open

The LP steam flow limitation also constrains the output capability of the repowered steam turbine. This is shown in the decrease in STG output from approximately 535 MW (nuclear) to 447 MW (repowered 6x1). Depending on the actual design of the steam turbine, it is possible that additional flow could be forced through the exhaust to allow more output, but this model uses the steam turbine heat balance exhaust flow as the limit. Capacity for a seventh CTG-HRSG train may be available. This would involve additional design work and an extensive evaluation of the PI steam turbine design.

Configuration Efficiencies

Trial performance runs of 2x1, 4x1, 4x1 with duct burners, and 6x1 configurations were made in the heat balance model. The results are presented in Table 2 below, Configuration Efficiencies of a Single Repowered Unit.

Table 2

Configuration Efficiencies of a Single Repowered Unit

Configuration	Net Plant Output (MW)	Net Plant Heat Rate (BTU/kwh, LHV)	STG Output (MW) (535 MW avail.)
2x1	450	6939	127
4x1	943	6815	280
4x1 with duct burners	1060	7310	401
6x1	1418	6599	447

Major Retained Equipment and Facilities

For the PI repower scenario, the following existing equipment and facilities were assumed to be available and incorporated into the cost model: STG, Condenser and Condensate System, Water Treatment System, Switchyard, Circulating Water System, Cooling Tower, Turbine Building, Administration Buildings.

Key Advantages

- Lower initial capital costs. The repowering option uses the most existing plant
 equipment. The repower option saves the process cost of a new STG, which
 according to the manufacturer is approximately \$35M FOB per STG at Prairie
 Island. With engineering and other costs, approximately \$100M in capital cost
 savings could be realized over a combined cycle plant.
- · Replaces baseload duty cycle of existing plant
- A repowered plant provides relatively efficient power if the conceptual design heat rate can be achieved. Note, however, that the existing steam turbine generators will not be optimized within a repowered steam cycle.
- If justifiable, an option exists to increase current site capacity by adding additional gas turbine generators from 4x1 to 6x1 or repowering the other plant.
- Gas turbines can be installed in simple cycle mode prior to full combined cycle mode to provide excess power or reduce the impact of the lost capacity.

Key Disadvantages

- Non-standardized design introduces uncertainties and longer installation cycles.
 These risks will be monetized by higher engineering fees, higher project
 contingency costs, and higher financing costs. For example, the Mystic project in
 Massachusetts, which is a first of a kind design in that it is the largest combined
 cycle plant in the US, is behind schedule and as of July 1, 2002, is experiencing
 hundreds of millions of dollars in cost overruns.
- Large natural gas capacity requirements and modifications.
- The attendant poorer reliability of older existing equipment retained in a repowered plant will likely result in higher maintenance costs over new equipment.

- The most optimal 6x1 repower configuration is not practical as it results in a plant output that will require switchyard modifications, cooling tower upgrades, and may require significant transmission system upgrades.
- Repowering in a phased construction approach to maintain continuity of site power output introduces significant regulatory uncertainty and risk if one nuclear unit is maintained operational. See Nuclear Issues section below.
- The repowered plant's duty cycle is not as flexible as that for a combined cycle unit.

Natural Gas Requirements

Discussion

Each scenario relies on a combustion turbine for power conversion. Consequently, the project must have access to a reliable high-pressure supply of natural gas. The combined cycle CTs will require significant volumes of gas provided on a 24-hour firm basis that will require capacity additions for the natural gas supplier. This involves a firm design load of approximately 200,000 mcf/day of natural gas for the combined cycle and single repower alternatives depending on the configuration and dispatch characteristics.

The simple cycle plants were assumed to require gas on a 5x16 summer operation protocol. Although gas pressures within interstate gas transmission lines are typically maintained above 1000 psig, the pressure levels maintained within the LDC's system are substantially lower (<100 psig) and are insufficient for proper operation of a large CT. Gas pressure within a distribution system is typically increased by adding compressor facilities, by enlarging or paralleling with existing high-pressure mains, and by constructing new supply mains. This results in significant additions to capital costs. For the purposes of this study, it was assumed that natural gas would be available at the site at sufficient pressure to eliminate the need for an onsite gas compressor.

In addition to equipment costs, the large gas loads associated with CT operation will require the supplier or a third party to actively manage the gas supply to maintain capacity and system integrity, which will tend to increase the plant O&M costs.

The two potential natural gas suppliers for the Red Wing Station are Viking Gas Transmission Company (Viking), an Xcel subsidiary, and Northern Natural Gas Company (Northern), formerly an Enron subsidiary now owned by Dynegy. On August 19th, Dynegy sold the Northern pipeline to MidAmerican Energy Holdings.

Viking

In order to supply gas to the PI site, Viking will need to install a 47-mile lateral line and a metering station. In addition, the mainline will have to be expanded to accommodate the high gas throughputs of the various plants. The capacity of the existing mainline is insufficient to supply the large gas load and this requires significant infrastructure modifications to increase system capacity. The mainline cost shown below is the up front

capital required to expand Viking's mainline to move the additional volumes from Emerson to the proposed lateral. Table 3 below shows a summary of Viking gas costs to support the various scenarios.

Table 3
Viking Gas Capital Costs (000s)

Plant Configuration	Lateral	Metering Station	Compression and Mainline Improvements	Total
Two Simple Cycle Replacement Units	\$29,870	\$430	\$262,000	\$292,300
Two Combined Cycle Replacement Units	\$25,620	\$275	\$176,000	\$201,895
One Repowered Unit (6x1)	\$29,870	\$350	\$220,000	\$250,220
Two Repowered Units	\$29,870	\$480	\$289,225	\$319,575

Northern Natural Gas

The Northern Natural Gas (NNG) system is physically closer to the PI site than the Viking system. The length of the lateral would be approximately 28 miles and would originate from the NNG Farmington compressor site. The NNG system is not as capacity constrained as the Viking pipeline and requires less mainline modifications to accommodate the proposed PI load. Table 4 below shows the Northern Natural Gas costs to support the various scenarios. Given the clear cost advantages, it was assumed that NNG would act as the project gas supplier.

Table 4 Northern Natural Gas Capital Costs (000s)

Plant Configuration	Lateral	Metering Station	Compression and Mainline Improvements	Total
Two Simple Cycle Units (interruptible)	\$28,000	\$600	NA	\$28,600
Two Simple Cycle Units	\$28,000	\$600	\$4100	\$32,700
Two Combined Cycle Units	\$22,700	\$600	\$4100	\$27,400
One Repowered Unit	\$28,000	\$600	\$4100	\$32,700
Two Repowered Units	\$34,600	\$800	\$5500	\$40,900

Water and Cooling Requirements

The Prairie Island Circulating Water System is appropriated 615 million gal/day of surface (river) water by DNR permit #69-072. The well water permits for PI allow consumption of approximately 470 gpm. This allotment is well in excess of the makeup and cooling water requirements of any of the above scenarios. A typical combined cycle plant uses on the order of 3 to 5 million gal/day.

A simple cycle plant does not require significant amounts of makeup water. The maximum consumption would be approximately 750 gpm (per 6 unit block) if the gas turbines were operated on fuel oil. This consumption rate is well within the existing water permit. With onsite storage tanks, the simple cycle plants could feasibly operate within the capacity provided by the well water only. If only natural gas is used fuel, only insignificant amounts of water would be required as water or steam injection for NOx control would not be necessary.

The existing circulating water system and associated cooling towers can be used as heat sinks for the proposed alternatives. Cooling towers are not required for the simple cycle plants.

Transmission Issues

The MW outputs of the power block configurations used in this study were chosen to match and fully utilize the existing transmission capability of the site. If the new generating equipment supplies power in excess of the capability and ratings of the existing switchyard and transmission system, such as in the 6x1 repower case, switchyard and transmission modifications will be needed. For the simple and combined cycle cases and the 4x1 single repower case, the output of the new units is within the existing switchyard ratings, and no significant switchyard modifications were assumed.

An interconnection study is necessary to determine the transmission system impact of the alternative generation. As part of the siting process, all new generation facilities are analyzed to determine the impact on the reliability of the associated electrical transmission study. These studies include analyses of fault duty, stability, and system voltage support. Usually fault duty studies are undertaken first. If these results are favorable, additional studies are conducted. An interconnection study must be requested through the Midwest ISO or developed by a third party. Generally ISO studies are undertaken when a certain project is likely to be developed, and the generation is likely to eventually become part of the system model. An ISO study cost is approximately \$40,000, depending on complexity. Since this feasibility study is preliminary and somewhat prospective in nature, interconnection studies were not performed.

NSP has examined thermal limitations for substation capacity increases for the 2001 All-Source Request for Supply Proposals. This indicative finding showed that approximately 800 MW could be added on the 345 KV bus at Prairie Island without exceeding loadings on transmission elements. Given this finding, all cases except the double repower case would not require mitigation for this particular facet of an interconnection study. It is very important to note, however, that the Prairie Island output is presently constrained by a flowgate on the Prairie Island-Byron interface such that no increases in capacity above the present capacity could be undertaken without system modifications.

Given these constraints and the increase in capacity above existing, the 6x1 repower configuration will require transmission and switchyard modifications and the double repower case will likely require transmission and switchyard modifications and additional modifications to demonstrate fault duty compliance. A full interconnection study is necessary to further evaluate feasibility and to determine more detailed cost estimates.

Nuclear Regulatory Issues

Natural Gas and Spent Fuel Interaction

There are two natural gas powered generation projects at former nuclear plant sites in the United States. These projects provide some insight into natural gas generation projects at Prairie Island. A repowering project at Fort St. Vrain (FSV) is complete and operational. A capacity replacement project at Rancho Seco is in the siting phase. Both of these projects involved previously decommissioned reactors with spent nuclear fuel completely transferred to an Independent Spent Fuel Storage Installation (ISFSI) prior to construction of the natural gas fired units. The repowering options at Prairie Island would involve evaluating the impact of large quantities of natural gas on site with spent nuclear fuel still located in the reactor or spent fuel storage pool.

Each of these projects was required to examine nuclear impacts to the spent fuel stored in the ISFSI. The NRC regards nuclear impacts as minimal as long as the new plant is greater than one half mile from the nuclear fuel and the new plant has been sufficiently isolated and secured from the existing nuclear plant. Gas and oil installations within ½ mile of an ISFSI require specific evaluations of the possible impacts to the nuclear fuel and prior NRC approval. This spatial isolation is a requirement of the ISFSI license at FSV. SMUD controls a large plat of land at the Rancho Seco site, and they were able to use the existing switchyard while locating the plant sufficiently far from the nuclear unit and the ISFSI. The SMUD project does not involve gas or oil impacts within ½ mile of the fuel. As of August 23, 2002, all of the Rancho Seco fuel was transferred to dry storage.

The ISFSI at FSV is located 1400 ft away from the nearest gas line. The NRC determined that this arrangement was satisfactory from a safety standpoint (FSV safety evaluation). This required examinations of the effects of postulated natural gas accidents. At FSV, the effects of a service line rupture, a main supply line rupture and a turbine building detonation were reviewed and found not to impact the safety function of the ISFSI.

Given the above, it would be in the nuclear safety and economic interests of a PI project to locate a natural gas power plant and supporting gas infrastructure at least one half mile from the fuel, whether the fuel is located in the spent fuel pool, the reactor, or the ISFSI. By examining the PI site layout, this appears at least geographically possible for the simple and combined cycle capacity replacement scenarios by locating these plants at the far northern boundary of the site. (Other analysis such as soil mechanics would have to be accomplished.) A gas line that is within ½ mile of an ISFI or a spent fuel pool does not, of itself, disqualify a project, but such a location will entail detailed failure mode and effects analyses for nuclear safety concerns.

The PI repower scenario that contemplates continued operation of one of the nuclear units during construction of a repowered unit entails significant regulatory uncertainty because of the safety ramifications of a failure mode and effects analyses. Repowering cannot be accomplished outside of the standard ½ mile interface area established by the NRC. The pressure drop between the HRSG superheater discharge and the existing steam turbine nozzle, which is a strong function of the length of the steam pipe run, should be minimized for plant efficiency.

There is no precedent that contemplates construction of a repowered plant that uses one of the two existing STGs at an operating nuclear power plant in the United States. High volume natural gas facilities introduce explosion hazards and safety concerns to an operating nuclear plant that would be hard to justify on a basis that repowering may have economic advantages over alternative generation. For instance, natural gas from a pipe failure could enter a structure through ventilation systems and be ignited and affect operators and nuclear safety equipment. Explosions have occurred at natural gas fired power plants. In 1999, a natural gas explosion destroyed a boiler at a KCPL coal plant. An explosion and large fire occurred at Sithe's South Boston 700 MW natural gas power plant on October 1, 2002.

Nuclear Safety and Project Reviews

It is estimated that from the time of a decision to pursue the repowering option that it would take approximately two years to complete the nuclear regulatory (NRC) review process. This two years includes 6 months for the licensee to prepare the required safety analyses for submittal, an estimated 6 months for review by the Nuclear Regulatory Commission and 1 year for public hearings should they be requested.

As part of the siting process, a repowering project would be subject to an analysis of feasible generation alternatives, which is required as part of the state's review to determine a given project's environmental impact. This would involve a review of the comparative merits of other reasonable alternatives to the repowering project that could satisfy the project objectives but may avoid or lessen the effects of the project. A competent reviewer would certainly need to examine the relative risks of repowering due to the proximity of nuclear fuel over other plausible alternatives such as siting replacement generation elsewhere. Because of the nuclear safety impacts, a favorable ruling for the repowering alternative, especially on a site with an operating nuclear plant,

over other generation alternatives may be difficult to obtain regardless of an NRC approval. For these and other reasons, a repowering project would likely be the subject of legal challenges from interveners. There are no industry precedents for siting a natural gas power plant on a nuclear site where the reactor has not been decommissioned. The ability to successfully license a repowered plant at Prairie Island cannot be predicted with any certainty. These feasibility risks should be well understood prior to undertaking a repowering project.

Environmental Considerations

For the purposes of this study, it was assumed that Best Available Control Technology (BACT) environmental controls are installed consistent with recent MPCA requirements for similar plants in attainment areas. For the combined cycle and repowering cases, it was assumed that dry low NOx combustion turbines and SCRs were installed.

The specific environmental impacts of routing the gas line or constructing and operating the plant have not been identified. The cost of the environmental surveys and consulting work has been included in the model. Environmental externalities were not monetized for this analysis. There are no cost provisions for environmental mitigation measures, such as purchasing wetlands for the purpose of set asides for compensatory habitat. These issues would be addressed in a more detailed study.

Continuity of Site Capacity

Transition Time

The scenarios addressed herein postulate a simultaneous shutdown of both nuclear units in the last quarter of 2006 followed by operation of the replacement or repowered units on or about January 2007. Current planning indicates a shutdown of Unit One in mid 2006 and Unit Two in late 2006 if additional spent fuel casks are not installed. For simplification purposes, the analysis assumes a simultaneous shutdown of both nuclear units such that the commercial operation of the gas-fired units is assumed to approximately coincide with the nuclear shutdown.

These cases, however, are somewhat hypothetical with regard to complete continuity of site power in that the integration and operation of the gas-fired units for continuous service would involve some modification and preparation of equipment formerly used by the nuclear unit(s) presumed to shutdown. Depending on regulatory requirements, the final routing of the gas pipeline onto the site may be scheduled subsequent to the nuclear plant shutdown. First fire of associated plant equipment would occur after the gas line had been installed. In addition, system and integrated plant testing would also need to be accomplished. For the purposes of this report this time will be referred to as the transition time. Transition time should be scheduled to occur when the impact to the grid is minimized much like a planned outage is scheduled. In general, the transition time would be a function of how much equipment is retained from the existing plant to the new plant. Detailed planning and staging equipment can minimize transition time. There

are, however, practical limits to optimizing this process because of the number of plant systems that need to be tested and certified for insurance, warranties, contractual requirements and other purposes.

Because of the uniqueness of this project, there are no direct examples available of transition time for a project of this type, but a reasonable estimate can be made from similar projects. A repower of a similar steam turbine at a fossil fueled plant (Ft. Myers) is expected to have a transition time of approximately 6 months. According to the EPRI model used for this study, the full testing phase of a typical combined cycle plant without nuclear complications is on the order of 7 months. Recent combined cycle projects have executed the testing phase in 4 to 6 months. Some have taken much longer. Given this information and allowing for nuclear-related contingencies, a reasonable estimate for transition time would be six months for a combined cycle replacement project and nine months for a repower of one unit. This estimate assumes that the NRC does not require any other additional testing or special requirements for nuclear safety purposes. If this occurs, which is not unlikely, the transition time will be extended, perhaps significantly.

Siting, Design, and Construction Times

Because of design standardization, combined cycle plants are being designed and constructed well within 3 years of a notice to proceed. Some combined cycle projects have been completed in 24 months or less. Simple cycle plants are less complex and can be completed in less time than combined cycle plants. The PI site has inherent advantages such as existing administrative buildings and other infrastructure that would contribute to a reduction in the construction time. The supporting off site natural gas infrastructure can be designed and constructed in 2 years and can be done in parallel with the power block design and construction. Allowing six months for up front siting work, no delays in regulatory approvals, and reasonable transition times (as defined above), the combined cycle and single unit repower generation alternatives could feasibly be completed by late 2006 if a decision is made by the second quarter 2003.

The timing of regulatory approvals for the repower cases, however, is subject to potentially lengthy delays due to siting issues and licensing uncertainty. A replacement simple or combined cycle plant that cannot be located outside of ½ mile from the area would also be subject to more detailed nuclear safety requirements and more uncertain regulatory approval times. See Nuclear Regulatory Issues above.

Phased Construction to Support an Extended Service Life of Nuclear Unit 2

The phased approach would involve a replacement of the retired capacity associated with the shutdown of one nuclear unit followed later by a replacement of the retired capacity associated with the shutdown of the second nuclear unit when the spent fuel pool is full. At the end of Phase 1, a gas-fired unit and a nuclear unit are providing power. At the end of Phase 2, two matching gas-fired units are providing power, and the nuclear units are retired. For the PI site this would involve an earlier shutdown of Unit One in fall of 2004 without initiation of its last fuel cycle in order to extend the service life of the Unit Two

by approximately 18 months to mid 2008 (depending on the fuel burnup rate). This is not considered feasible or desirable for reasons discussed below.

It is not realistic to assume that a combined cycle or repowered plant can be fully completed by the fall of 2004. A simple cycle plant or the simple cycle portion of a combined cycle plant could possibly be completed if the project is authorized and notice to proceeds for various contracts are issued by early 2003 and no delays in siting, design, procurement, and construction, including natural gas infrastructure, are experienced. The combined cycle portion could be finished by early 2006. Given the unique nature of this project where the siting and construction necessarily involves a first-of-a-kind review of the impacts to an operating nuclear power plant (Unit Two), a streamlined fast track process with no delays is considered extremely unlikely.

A phased approach will cost more (estimate 30%-50%) because: 1) the Engineer Procure and Construct (EPC) contractor will require contingencies and incentives to complete the complex project on an abbreviated schedule, and 2) resources are mobilized at two different times as the second natural gas generation unit is completed years later from the first plant. This approach does provide some flexibility in that it sets up an option to cancel construction of the second unit if system load decreases or if other substitute generation capacity is added. If a phased construction approach for repowering were undertaken, the combustion turbines could be installed in increments, however the work available from the turbine would not be as efficiently utilized until all six CTs were installed. As discussed above, at interim gas turbine configurations the net plant output will decrease and the plant heat rate will degrade somewhat at configurations less than a 6x1 (2x1, 4x1). This approach would also cost more than an uninterrupted project.

Another consideration is that the cost to maintain a nuclear plant shutdown without a possession only license (which can be obtained from the NRC post decommissioning) can easily be as much or more as that needed to maintain it operating. Because of relatively inexpensive fuel costs, the variable operating costs at nuclear units are much less than those of a fossil unit. Because of higher labor, shutdown maintenance, and insurance costs, the fixed costs for a shutdown nuclear unit are significant. Finally, the economies of scale that are realized with both units operating would be lost.

Stand Alone Construction

In order to minimize impacts to the existing nuclear plant, the simple and combined cycle plants could be designed and built without the use of any existing site power equipment. Administrative buildings and non-safety related infrastructure could still be used. This would add approximately \$20 million of equipment costs to the simple cycle plant and \$50 million to the combined cycle plants. Of course this is not an option for the repowering alternatives. One potential feasibility risk element with this approach is that it would involve changes to the surface water appropriation and the existing circulating water system. These changes engender a much more expensive and less streamlined approach to the siting process due to the necessity to obtain changes in the plant water permits.

In addition, once stand-alone construction is contemplated, a competent generation planner would compare the costs of stand alone construction at PI with a greenfield generation project at carefully chosen offsite location. It is altogether likely that the greenfield site offsite would pose significantly less risk and also be price competitive with a stand alone project at PI.

Schedule

A representative Level 1 schedule has been provided in the appendix that shows an estimate of the power plant development and construction cycle to satisfy a 2007 startup. It was assumed the gas pipeline projects could be completed in parallel with the design and construct power plant tasks without affecting the critical path elements. According to Northern Natural Gas, a general time estimate for the design and FERC filing requirements for a project of this scope is one year. An in-service construction timeframe estimate for a project of this scope would also be approximately one year. There likely would be some overlap in these time horizons such that a reasonable project timeline estimate to complete the gas pipeline project would be 1.75 years.

Operations and Maintenance (O&M) Costs

The fixed and variable O&M costs for each practical scenario is given in Table 5, Alternatives Operations and Maintenance Costs below. Gas costs, which are highly volatile, were not included in the O&M estimates. The fixed O&M costs do not include any future capital upgrades. The variable costs assume 10% capacity factor for simple cycle and a 92% capacity factor for combined cycle and repower. These costs also do not include any costs to operate, maintain, demolish, or provide security for any of the PI nuclear facilities.

Table 5
Alternatives Operations and Maintenance Costs

Alternative	Fixed O&M (\$/kw-yr)	Non-gas Variable O&M (\$/MWh)
Simple Cycle Capacity Replacement	2.37	2.67*
Combined Cycle Capacity Replacement	3.23	1.79**
Repower One Unit with Duct Burners	3.15	1.68**
* 10% capacity factor ** 92% capacity factor		

Appendix

Schedule Combined Cycle or Repower Plant

Schedule	Planned	Planned	Planned
	Start	End	Duration (Days)
Design, Procurement and Delivery	1/1/2004	9/1/2006	974
Engineering	1/1/2004	3/1/2006	790
Permitting	1/1/2004	6/1/2005	517
Procurement, Fabrication and Delivery	4/1/2004	9/1/2006	883
Construction	4/15/2005	11/1/2006	565
Mobilization and Site Preparation	4/15/2005	6/1/2005	47
Underground Piping, Elec and Misc Facilities	6/15/2005	3/15/2006	273
Field Erected Tanks	10/1/2005	2/1/2006	123
Substructure Work	5/15/2005	10/15/2005	153
Superstructure Work	1/1/2006	6/1/2006	151
HRSGs and Aux Installation	6/15/2005	8/1/2006	412
Combustion Turbine Installation	11/1/2005	9/1/2006	304
Steam Turbine Installation*	1/1/2006	6/1/2006	151
Balance Of Plant (BOP) Equip Installation	12/15/2005	10/15/2006	304
BOP Electrical Sys Installation	12/15/2005	10/15/2006	304
BOP Control and Instrumentation Installation	1/1/2006	11/1/2006	304
Final Site and Finish Architectural Work	8/1/2006	11/1/2006	92
Testing	4/1/2006	1/1/2007	275
Plant Startup	4/1/2006	12/1/2006	244
Combustion Turbine Startup	5/1/2006	10/1/2006	153
HRSG Startup	5/15/2006	11/1/2006	170
Steam Turbine Startup	4/15/2006	12/1/2006	230
Plant Performance Testing	12/1/2006	1/1/2007	31
Commercial Operating Date	1/1/2007	1/1/2007	
* Steam turbine integration for repower case			

Cost and Emission Data

SIMPLE CYCLE

SIMPLE CYCLE COSTS

TOTAL PROCESS CAPITAL	364,105,984	
General Facilities	14,564,240	
Engineering and Home Office Fees	25,487,420	
Project Contingency	36,410,600	
Process Contingency	0	
TOTAL PLANT COST	440,568,256	
AFUDC or IDC		
See Capital Outlay Table		
TOTAL PLANT INVESTMENT	440,568,256	
Prepaid Royalties	0	
Preproduction Costs	18,875,486	
Inventory Capital	2,202,841	
Initial Cost - Catalyst and Chemicals	0	
Land	0	
Capital Cost Adders	32,700,000	
TOTAL CAPITAL REQUIREMENT	494,346,592	
TOTAL CAPITAL REQUIREMENT (Currency/net kW)		494.8
O + M and Fuel Costs		
(in Base Year (2002) Currency)		
Fixed O + M		
Direct Operating Labor	406,140	
- Number of Operating Staff	5	
Direct Maintenance Labor	519,770	
- Number of Maintenance Staff	9	
Annual Services, Materials, & Purchased Power		
- Annual O&M Services & Materials	552,259	
- Non-operating Purchased Power	397,264	
Indirect Labor Costs		
- Benefits	273,404	
- Home Office Costs	216,486	
TOTAL FIXED O+M	2,365,325	

SIMPLE CYCLE COSTS (Continued)

Variable O+M		
Scheduled Maintenance Parts & Materials		
- CT Inspection/Overhaul	1,998,717	
- HRSG Inspection/Refurbish	0	
- ST Inspection/Overhaul	0	
- BOP Refurbish	20,876	
Scheduled Maintenance Labor		
- CT Inspection/Overhaul	139,910	
- HRSG Inspection/Refurbish	0	
- ST Inspection/Overhaul	0	
- BOP Refurbish	32,861	
Unscheduled Maintenance Allowance	109,618	
Catalyst Replacement		
- SCR Catalyst Materials & Labor	0	
- CO Catalyst Materials & Labor	0	
Other Consumables		
- Raw water	11,830	
- Circulating water	0	
- NH3	0	
- H2SO4	12,979	
- NaOH	15,673	
- Misc	15,968	
Disposal Charges		
- Spent SCR catalyst	0	
- Spent CO catalyst	0	
- Other disposal	75	
Byproduct Credit	0	
Total Variable O+M	2,358,510	
Total Variable O+M (Currency/MWh)		2.67
Total Fixed and Variable O+M	4,723,835	
Fuel Cost	20120-110	
Fuel Cost	31,934,028	00.47
Fuel Cost (Currency/MWh)		36.17

SIMPLE CYCLE CAPITAL OUTLAY

Category	Total	1	2	3
Calendar Year (Jan 1 - Dec 31)		2004	2005	2006
Tatal Blood Cook				
Total Plant Cost In Base Year (2002) Currency	440,568,256	9.862.531	162,282,496	268,423,232
	32,458,140			22,126,706
Amount of Escalation Escalated Total Plant Cost	473,026,432		172,215,488	
	23,042,888			23,042,888
Other Outlays(*)	496,069,312		172,215,488	
Gross Outlay	430,003,512		0	0
Investment Tax Credits	0	0	0	0
Other Income Tax Offsets	U	· ·		
Net Total Capital Requirement	400 000 240	10 000 077	170 015 400	212 502 832
Net Cash Outlay	496,069,312		172,215,488	313,332,002
AFUDC - Equity(**)	26,179,826			
AFUDC - Interest	16,696,738			
Total (Excluding capital cost adders)	538,945,856			
Gross Depreciable Investment		510,357,920		
Non-Depreciable Net Plant Outlay(***)	2,408,152			
Equity AFUDC	26,179,826			
Total Non-Depreciable Investment		28,587,978		
Capital Cost Adders	32,700,000			
Total Capital Requirement	957(9 5 5) 3 0 5	571,645,888		
Less Investment Tax Credit		0		
Net Total Capital Requirement		571,645,888		
(*) Consists Of				
Land	0			
Preproduction Costs	20,634,736			
Prepaid Royalties	0			
Inventory Cap + Init Cat/Chem	2,408,152			
Total	23,042,888			
(**) Consists of:				
Preferred Stock AFUDC				
	26,179,826			
Common Equity AFUDC Total	26,179,826			
(***) Consists of:				
Land	0			Gaz
Inventory Cap + Init Cat/Chem	2,408,152			
INVENTORY CAN T THE CALCHETT	4,400,104			

SIMPLE CYCLE EMISSIONS

Variable	Value	Units
PLANT DESIGN BASIS		
Ambient Air Temperature	59	F
Site Elevation Above MSL	695	ft
Cycle Type	Simple Cycle	
Number of Combustion Turbines Operating	12	
CT Primary Fuel Type	Natural Gas	
CT NOx Control Type - Primary Fuel	Dry Low NOx Combustors	
Inlet Air Cooling	Fogging	
CT Air Precooler Discharge Temperature	52	F
AIR EMISSIONS - COMBUSTION TURBINES	3	
Firing Primary Fuel		
CO2 Mass Flow Per CT Stack	113,904.96	lb/h
CO Mass Flow Per CT Stack	53.27	lb/h
NOx (As NO2) Mass Flow Per CT Stack	31.51	lb/h
SO2 Mass Flow Per CT Stack	0	lb/h
CO Concentration	25	ppmvd @ 15% O2
NOx Concentration	9	ppmvd @ 15% O2
SO2 Concentration	0	ppmvd @ 15% O2
Volumetric Flow Rate Per CT Stack	1,483,875	ft3/min-act
CO2 Mass Flow Total Plant	1,366,859.50	lb/h
CO Mass Flow Total Plant	639.24	lb/h
NOx (As NO2) Mass Flow Total Plant	378.07	lb/h
SO2 Mass Flow Total Plant	0	lb/h
LIQUID DISCHARGES		
Total Waste Water Discharge Peak Flow	962	gpm
Total Waste Water Discharge Average Flow	29	gpm

COMBINED CYCLE

COMBINED	CVCI	E	COST	2
L. CHALLESTAN INC.	LITTLE		CUSI	u

TOTAL PROCESS CAPITAL	452,102,016	
Constant Facilities	13,563,060	
General Facilities	31,647,140	
Engineering and Home Office Fees	45,210,200	
Project Contingency	0	
Process Contingency		
TOTAL PLANT COST	542,522,368	
AFUDC or IDC		
See Capital Outlay Table		
TOTAL PLANT INVESTMENT	542,522,368	
TOTAL PLANT INVESTMENT (\$/kW)		515.02
Prepaid Royalties	0	
Preproduction Costs	17,795,884	
Inventory Capital	2,712,611	
Land	0	
Capital Cost Adders	27,400,000	
TOTAL CAPITAL REQUIREMENT	590,430,848	
O + M and Fuel Costs		
(in Base Year (2002) \$)		
Fixed O + M		
Direct Operating Labor	1,069,159	
- Number of Operating Staff	17	
Direct Maintenance Labor	901,818	
- Number of Maintenance Staff	15	
Annual Services, Materials, & Purchased Power		
- Annual O&M Services & Materials	348,525	
- Non-operating Purchased Power	115,347	
Indirect Labor Costs		
- Benefits	616,896	
- Home Office Costs	294,421	-No
TOTAL FIXED O+M	3,346,168	

COMBINED CYCLE COSTS (continued)

Variable O+M		
Scheduled Maintenance Parts & Materials		
- CT Inspection/Overhaul	9,312,600	
- HRSG Inspection/Refurbish	592,303	
- ST Inspection/Overhaul	744,000	
- BOP Refurbish	500,000	
Scheduled Maintenance Labor		
- CT Inspection/Overhaul	651,882	
- HRSG Inspection/Refurbish	177,691	
- ST Inspection/Overhaul	111,000	
- BOP Refurbish	85,199	
Unscheduled Maintenance Allowance	582,049	
Catalyst Replacement		
- SCR Catalyst Materials & Labor	177,024	
- CO Catalyst Materials & Labor	0	
Other Consumables		
- Raw water	1,831,258	
- Circulating water	0	
- NH3	50,773	
- H2SO4	39,568	
- NaOH	47,780	
- Misc	44,655	
Disposal Charges		
- Spent SCR catalyst	11,064	
- Spent CO catalyst	0	
- Other disposal	3,875	
Byproduct Credit	0	
Total Non Gas Variable O+M	14,962,721	
Total Non Gas Variable O+M (\$/MWh) 92% CF		1.83
Total Fixed and Variable O+M	18,308,889	

COMBINED CYCLE CAPITAL OUTLAY

Category	Total	1	2.4.2.2	
Calendar Year (Jan 1 - Dec 31)		2004	2005	2006
Total Plant Cost				
In Base Year (2002) Currency	516,219,648	21,360,704	57,966,872	436,892,064
Amount of Escalation	40,424,964	862,972	3,548,036	36,013,956
Escalated Total Plant Cost	556,644,608	22,223,676	61,514,908	472,906,016
Other Outlays(*)	21,705,646	0	0	21,705,646
Gross Outlay	578,350,208	22,223,676	61,514,908	494,611,648
Investment Tax Credits	0	0	0	C
Other Income Tax Offsets	0	0	0	0
Net Total Capital Requirement				
Net Cash Outlay	578,350,208	22,223,676	61,514,908	494,611,648
AFUDC - Equity(**)	23,202,630			
AFUDC - Interest	14,858,861			
Total (Excluding capital cost adders)	616,411,712			
Gross Depreciable Investment		590,387,456		
Non-Depreciable Net Plant Outlay(***)	2,821,663			
Equity AFUDC	23,202,630			
Total Non-Depreciable Investment		26,024,294		
Capital Cost Adders	27,400,000			
Total Capital Requirement	1,511,012,014	643,811,776		
Less Investment Tax Credit		0		
Net Total Capital Requirement		643,811,776		
(*) Consists Of				
Land	0			
Preproduction Costs	18,883,982			
Prepaid Royalties	0			
Inventory Cap + Init Cat/Chem	2,821,663			
Total	21,705,646			
(**) Consists of:				
Preferred Stock AFUDC	0			
Common Equity AFUDC	23,202,630			
Total	23,202,630			
(***) Consists of:				
Land	0			1.00
Inventory Cap + Init Cat/Chem	2,821,663			
Total	2,821,663			

COMBINED CYCLE EMISSIONS		
Variable	Value	Units
PLANT DESIGN BASIS		
Ambient Air Temperature	59	F
Site Elevation Above MSL	695	ft
Cycle Type	Combined Cycle Cogeneration	
Number of Combustion Turbines Operating	4	
CT Primary Fuel Type	Natural Gas	
CT NOx Control Type - Primary Fuel	Dry Low NOx Combustors	
CT Air Precooler Discharge Temperature	59	F
Cooling System Type	Wet Mech Draft Cooling Twr	
SCR Configuration	Anhydrous Ammonia Injection	
NOx Conversion Efficiency (%), Primary Fuel	45	%
AIR EMISSIONS - HRSG's		
Firing Primary Fuel		
CO2 Mass Flow Per HRSG Stack	213,608.19	lb/h
CO Mass Flow Per HRSG Stack	40 42	lb/h
NOx (As NO2) Mass Flow Per HRSG Stack	33.2	lb/h
NH3 Mass Flow Per HRSG Stack	12.27	lb/h
SO2 Mass Flow Per HRSG Stack	0	lb/h
CO Concentration	10	ppmvd @ 15% O2
NOx Concentration	5	ppmvd @ 15% O2
NH3 Concentration	5	ppmvd @ 15% O2
SO2 Concentration	0	ppmvd @ 15% O2
Volumetric Flow Rate Per HRSG Stack	1,045,319	ft3/min-act
CO2 Mass Flow Total Plant	854,432.75	1b/h
CO Mass Flow Total Plant	161.68	lb/h
NOx (As NO2) Mass Flow Total Plant	132.81	lb/h
NH3 Mass Flow Total Plant	49.08	lb/h
SO2 Mass Flow Total Plant	0	lb/h
LIQUID DISCHARGES		
Raw Cycle Water Make-up Peak Flow	147	gpm
Raw Cycle Water Make-up Average Flow	98	gpm
Cooling Tower Make-up Peak Flow	7,319	gpm
Cooling Tower Make-up Average Flow	4,879	gpm
Cooling Tower Blowdown Peak Flow	1,403	gpm
Cooling Tower Blowdown Average Flow	936	gpm
Total Waste Water Discharge Peak Flow	18,747	gpm
Total Waste Water Discharge Average Flow	1,036	gpm
SOLID WASTES		
SCR Catalyst Material	Vanadium Pentoxide/Zeolite	
SCR Catalyst Volume	922	ft3
SCR Catalyst Replacement Frequency	5 to 10	years

REPOWER

REPOWER	ONE	FIRST	AVI	Caete
KEPUVVEK	UNE	UNII	401	COSIS

TOTAL PROCESS CAPITAL	342,284,992	
General Facilities	10,268,550	
Engineering and Home Office Fees	23,959,950	
Project Contingency	34,228,500	
Process Contingency	0	
TOTAL PLANT COST	410,742,016	
AFUDC or IDC		
See Capital Outlay Table		
TOTAL PLANT INVESTMENT	410,742,016	
TOTAL PLANT INVESTMENT (\$/kW)		386
Prepaid Royalties	0	
Preproduction Costs	15,530,286	
Inventory Capital	2,053,709	
Initial Cost - Catalyst and Chemicals	0	
Land	0	
Capital Cost Adders	37,400,000	
TOTAL CAPITAL REQUIREMENT	465,725,984	
O + M and Fuel Costs		
(in Base Year (2002) \$)		
Fixed O + M		
Direct Operating Labor	1,069,159	
- Number of Operating Staff	17	
Direct Maintenance Labor	901,818	
- Number of Maintenance Staff	15	
Annual Services, Materials, & Purchased Power		
- Annual O&M Services & Materials	374,337	
- Non-operating Purchased Power	120,404	
Indirect Labor Costs		
- Benefits	616,896	
- Home Office Costs	294,421	

TOTAL FIXED O+M

REPOWER ONE UNIT Costs (Continued)

Variable O+M		
Scheduled Maintenance Parts & Materials		
- CT Inspection/Overhau!	8,863,800	
- HRSG Inspection/Refurbish	582,614	
- ST Inspection/Overhaul	744,000	
- BOP Refurbish	500,000	
Scheduled Maintenance Labor	7.00	
- CT Inspection/Overhaul	620,466	
- HRSG Inspection/Refurbish	174,784	
- ST Inspection/Overhaul	111000	
- BOP Refurbish	85,199	
Unscheduled Maintenance Allowance	529,098	
Catalyst Replacement		
- SCR Catalyst Materials & Labor	172,608	
- CO Catalyst Materials & Labor	0	
Other Consumables		
- Raw water	1,843,527	
- Circulating water	0	
- NH3	46,357	
- H2SO4	39,547	
- NaOH	47,755	
- Misc	44,781	
Disposal Charges		
- Spent SCR catalyst	10,788	
- Spent CO catalyst	0	
- Other disposal	3,698	
Byproduct Credit	0	
Total Non Gas Variable O+M	14,420,022	
Total Non Gas Variable O+M (\$/MWh)		1.68
Total Fixed and Variable O+M	14,420,022	

REPOWER ONE UNIT Capital Outlay

Category	Total	1	2	
Calendar Year (Jan 1 - Dec 31)		2004	2005	2006
Total Plant Cost				
In Base Year (2002) Currency	394,110,016	16,382,303	53,702,056	324,025,664
Amount of Escalation	30,658,974	661,845	3,286,995	26,710,134
Escalated Total Plant Cost	424,769,024	17,044,148		350,735,808
Other Outlays(*)	18,576,800	0	0	18,576,800
Gross Outlay	443,345,792	17,044,148	56,989,052	369,312,608
Investment Tax Credits	0	0	0	0
Other Income Tax Offsets	0	0	0	0
Net Total Capital Requirement				
Net Cash Outlay	443,345,792	17,044,148	56,989,052	369,312,608
AFUDC - Equity(**)	18,400,370			
AFUDC - Interest	11,775,106			
Total (Excluding capital cost adders)	473,521,280			
Gross Depreciable Investment		452,966,720		
Non-Depreciable Net Plant Outlay(***)	2,154,211			
Equity AFUDC	18,400,370			
Total Non-Depreciable Investment		20,554,580		
Capital Cost Adders	37,400,000			
Total Capital Requirement		510,921,312		
Less Investment Tax Credit		0		
Net Total Capital Requirement		510,921,312		
(*) Consists Of				
Land	0			
Preproduction Costs	16,422,588			
Prepaid Royalties	0			
Inventory Cap + Init Cat/Chem	2,154,211			
Total	18,576,800			
(**) Consists of:				
Preferred Stock AFUDC	0			
Common Equity AFUDC	18,400,370			
Total	18,400,370			
(***) Consists of:				
Land	0			*
Inventory Cap + Init Cat/Chem	2,154,211			
Total	2,154,211			

REPOWER ONE UNIT 4X1 Emissions		
Variable	Value	Units
PLANT DESIGN BASIS		
Ambient Air Temperature	59	F
Site Elevation Above MSL	695	ft
Cycle Type	Combined Cycle Cogeneration	1
Number of Combustion Turbines Operating	4	
CT Primary Fuel Type	Natural Gas	
CT NOx Control Type - Primary Fuel	Dry Low NOx Combustors	
CT Air Precooler Discharge Temperature	59	F
Cooling System Type	Wet Mech Draft Cooling Twr	
SCR Configuration	Anhydrous Ammonia Injection	
NOx Conversion Efficiency (%), Primary Fuel	51	%
Include Duct Burners	Yes	
Duct Burner Use	Full-Time	
DB Primary Fuel Type	Natural Gas	
	1,000,00	
AIR EMISSIONS - HRSG's		
Firing Primary Fuel		
CO2 Mass Flow Per HRSG Stack	225,714.88	fb/h
CO Mass Flow Per HRSG Stack	57.35	lb/h
NOx (As NO2) Mass Flow Per HRSG Stack	34.77	lb/h
NH3 Mass Flow Per HRSG Stack	12.85	lb/h
SO2 Mass Flow Per HRSG Stack	0	lb/h
CO Concentration	14	ppmvd @ 15% O2
NOx Concentration	5	ppmvd @ 15% O2
NH3 Concentration	5	ppmvd @ 15% O2
SO2 Concentration	0	ppmvd @ 15% O2
Volumetric Flow Rate Per HRSG Stack	979,280	ft3/min-act
CO2 Mass Flow Total Plant	902,859.50	lb/h
CO Mass Flow Total Plant	229	lb/h
NOx (As NO2) Mass Flow Total Plant	139 07	lb/h
NH3 Mass Flow Total Plant	51.4	lb/h
SO2 Mass Flow Total Plant	0	lb/h
LIQUID DISCHARGES		
Raw Cycle Water Make-up Peak Flow	179	gpm
Raw Cycle Water Make-up Average Flow	119	gpm
Cooling Tower Make-up Peak Flow	8,681	gpm
Cooling Tower Make-up Average Flow	5,787	gpm
Cooling Tower Blowdown Peak Flow	1,664	gpm
Cooling Tower Blowdown Average Flow	1,110	gpm "
Total Waste Water Discharge Peak Flow	21,951	gpm
Total Waste Water Discharge Average Flow	1,231	gpm
A STATE OF THE PROPERTY OF THE PARTY OF THE		

REPOWER ONE UNIT 4X1 Emissions (Cont.)

SOLID WASTES SCR Catalyst Material SCR Catalyst Volume SCR Catalyst Replacement Frequency

Vanadium Pentoxide/Zeolite 1,039 ft3 37,386 years

New Mexico-Vacate Cicense 3/27/24

United States Court of Appeals for the Fifth Circuit united St

United States Court of Appeals Fifth Circuit

FILED

No. 23-60377 Summary Calendar March 27, 2024 Lyle W. Cayce Clerk

FASKEN LAND AND MINERALS, LIMITED; PERMIAN BASIN LAND AND ROYALTY OWNERS,

Petitioners,

versus

NUCLEAR REGULATORY COMMISSION; UNITED STATES OF AMERICA,

Respondents.

Appeal from the Nuclear Regulatory Commission Agency No. 72-1051

Before Jones, Elrod, and Wilson, Circuit Judges. Per Curiam:*

In September 2021 the Nuclear Regulatory Commission (NRC) issued a license to Interim Storage Partners, LLC, to establish a facility to store nuclear waste temporarily in Andrews County, Texas. See Texas v. Nuclear Regul. Comm'n, 78 F.4th 827, 833-35 (5th Cir. 2023) [hereinafter Texas v. NRC], reh'g en banc denied, 2024 WL 1108700 (5th Cir. Mar. 14,

^{*} This opinion is not designated for publication. See 5TH CIR. R. 47.5.

No. 23-60377

2024). Texas, Fasken Land and Minerals, Ltd., (Fasken), and Permian Basin Land and Royalty Owners (PBLRO) petitioned this court to set aside that license. *Id.* at 834–35. In that appeal, a panel of this court first held that Fasken and PBLRO had standing under the Constitution and the Hobbs Act to challenge the NRC's actions. *Id.* at 835–40. It then held that the NRC lacked statutory authority to issue the license. *Id.* at 840–44. Accordingly, this court granted the petitions for review and vacated the license. *Id.* at 844. The NRC filed a petition for rehearing *en banc* on October 24, 2023, which this court denied on March 14, 2024. *See Texas v. Nuclear Regul. Comm'n*, No. 21-60743, --- F.4th ----, 2024 WL 1108700 (5th Cir. Mar. 14, 2024).

Shortly before the panel issued its opinion in *Texas v. NRC*, Fasken and PBLRO filed the petition for review at issue in this case. They challenge a different license issued by the NRC in May 2023 to Holtec International to establish a facility to store nuclear waste in Lea County, New Mexico. The parties, correctly, agree that *Texas v. NRC* involved a "materially identical license in a materially identical procedural posture" and that "absent the [c]ourt granting rehearing en banc in *Texas* [v. NRC]..., the panel's consideration of this case will be controlled by [*Texas v. NRC*]." Because this court's holding in *Texas v. NRC* dictates the outcome here, we GRANT Fasken's and PBLRO's petition for review and VACATE the Holtec license. The NRC's motion to transfer the petition for review to the United States Court of Appeals for the District of Columbia Circuit is DENIED AS MOOT.

BUSINESS

The real cause of Xcel's lengthy Prairie Island nuclear outage: Workers drilled through cables

The company said it did not use ground radar that would have shown where the underground cables were in the Red Wing plant and admitted its excavation planning and oversight was inadequate.

By Walker Orenstein (https://www.startribune.com/walker-orenstein/11331478/) Star Tribune

APRIL 12, 2024 — 8:02AM

Xcel Energy initially pinpointed the cause of a lengthy outage at its Prairie Island nuclear power plant <u>as an equipment issue (https://www.startribune.com/xcels-prairie-island-nuclear-plant-will-be-out-of-commission-until-january/600321701/?refresh=true)</u> between the turbine and the electric grid.

But that didn't quite tell the full story: Xcel workers at the plant actually cut a bundle of power cables when drilling sideways underground in October, interrupting power to some of the Red Wing plant's equipment and causing one of the two reactors to shut down.

Xcel told federal nuclear regulators last month it did not use ground radar in an area that would have shown the cables' location. The company also said its excavation planning and oversight was inadequate, admitting to "procedural weaknesses and poor communications" between departments.

"This resulted in work progressing in the field without all controls in place that would be expected for work at a nuclear plant," an Xcel filing with the U.S. Nuclear Regulatory Commission read.



Dave Lochbaum, a nuclear safety engineer formerly with the Union of Concerned Scientists, said the plant's systems worked as intended when the reactor safely shut down. But he said cutting into power cables could have been a safety risk for the workers, which Xcel acknowledged as well.

Prairie Isla 29 7

"We know that any electrical contact can potentially be hazardous," Xcel spokesman Kevin Coss said. "None of the workers operating the drilling equipment were hurt when it impacted the cables. To prevent similar issues in the future, we have made changes to how we approach excavation projects as well as our oversight of workers coming to the plant for projects."

In April, Xcel told the Minnesota Department of Commerce it realized the damaged cables were aging and at risk of water damage after inspecting them. That would have likely caused both units at Prairie Island to shut down in the future, Xcel said.

ADVERTISEMENT

In March, Commerce also asked why Xcel customers should pay for the Unit 1 outage. In regulatory filings, the company said customers should cover the cost of fuel and power it bought on the energy market in place of nuclear energy during the Unit 1 outage because, despite causing the problem, Xcel was operating prudently.

"Prudence does not mean perfection, and even prudent decisions and operations can sometimes result in undesirable outcomes," Xcel wrote.

Unit 1 at Prairie Island went down on Oct. 19. That caused a longer-than-expected outage at the plant's second unit, too, which had powered down two weeks earlier for scheduled refueling and maintenance.

Xcel expected both reactors to be up and running in January, but the company <u>did not begin running both units at the plant again consistently until mid March (https://www.startribune.com/xcel-energy-prairie-island-nuclear-power-plant-puc-utility-commerce/600352107/)</u>.

The Xcel workers drilling underground were helping to replace a different power cable between an electric substation and the plant. The company replaced the damaged cables

rather than repair them. Xcel also took time when the plant was offline to do a more thorough plant inspection than it could have otherwise, among other maintenance, which will avoid the need to shut down both units in the future for that work.

The <u>aging nuclear plants</u> are a crucial part of Xcel's plans for a carbon-free energy grid by 2040 (https://www.startribune.com/xcel-aging-nuclear-plants-risks-monticello-prairie-island-red-wing-minnesota-clean-energy-2040-plan/600282822/). Nuclear makes up about 30% of the utility's power supply in the Upper Midwest.

Xcel wants to extend the life of its two units at Prairie Island (https://www.startribune.com/xcel-energy-long-term-plan-prairie-island-nuclear-gas-plants-wind-solar-large-scale-battery/600340390/) so they can run until 2053 and 2054, 20 years beyond when their current licenses expire, something that requires state and federal approval. Xcel has also asked the Public Utilities Commission for permission to store more nuclear waste at the site (https://www.startribune.com/xcel-prairie-island-plant-more-nuclear-storage/600342090/) in above-ground casks.

Walker Orenstein covers energy, natural resources and sustainability for the Star Tribune. Before that, he was a reporter at MinnPost and at news outlets in Washington state.

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TEXAS - Vacate license 8/25/23

United States Court of Appeals

for the Fifth Circuit

United States Court of Appeals Fifth Circuit

FILED

August 25, 2023

Lyle W. Cayce Clerk

No. 21-60743

STATE OF TEXAS; GREG ABBOTT, Governor of the State of Texas; Texas Commission on Environmental Quality; Fasken Land and Minerals, Limited; Permian Basin Land and Royalty Owners,

Petitioners,

versus

NUCLEAR REGULATORY COMMISSION; UNITED STATES OF AMERICA,

Respondents.

Appeal from the Nuclear Regulatory Commission Agency No. 72-1050

Before Jones, Ho, and WILSON, Circuit Judges.

JAMES C. Ho, Circuit Judge:

Nuclear power generation produces thousands of metric tons of nuclear waste each year. And such waste has been accumulating at nuclear power plants throughout the United States for decades. Congress has mandated that such waste be permanently stored in a geologic repository. But the development, licensing, and construction of that repository has stalled.

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To address this problem, the Nuclear Regulatory Commission has asserted that it has authority under the Atomic Energy Act to license temporary, away-from-reactor storage facilities for spent nuclear fuel. Based on that claim of authority, the Commission has issued a license for Interim Storage Partners, LLC, a private company, to operate a temporary storage facility on the Permian Basin, in Andrews County, Texas. Fasken Land and Minerals, Ltd., a for-profit organization working in oil and gas extraction, and Permian Basin Land and Royalty Owners ("PBLRO"), an association seeking to protect the interests of the Permian Basin, have petitioned for review of the license. So has the State of Texas, which argues, *inter alia*, that the Atomic Energy Act doesn't confer authority on the Commission to license such a facility.

Texas is correct. The Atomic Energy Act does not confer on the Commission the broad authority it claims to issue licenses for private parties to store spent nuclear fuel away-from-the-reactor. And the Nuclear Waste Policy Act establishes a comprehensive statutory scheme for dealing with nuclear waste generated from commercial nuclear power generation, thereby foreclosing the Commission's claim of authority. Accordingly, we grant the petition for review and vacate the license.

I.

This case is the latest development in a decades-long debate over nuclear power and waste regulation. Accordingly, we provide a brief overview of relevant historical and technical background before delving into the specifics of the licensing proceedings challenged here.

¹ For the remainder of this opinion, we use the term "Fasken" to refer to Fasken Land and Minerals, Ltd. and PBLRO collectively, unless addressing an issue where it's necessary to distinguish them.

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A.

The United States began producing nuclear waste in the 1940s, first as a byproduct of nuclear weapons development and then as a byproduct of the commercial nuclear power industry. Blue Ribbon Commission on America's Nuclear Future, Report to the Secretary of Energy 19 (Jan. 2012) https://www.energy.gov/sites/prod/files/2013/04/f0/brc_finalreport_jan2012.pdf [hereinafter BRC Report]. The first nuclear reactor was demonstrated in 1942, and Congress authorized civilian application of atomic power through the Atomic Energy Act of 1946. Pac. Gas & Elec. Co. v. State Energy Res. Conservation & Dev. Comm'n, 461 U.S. 190, 206 (1983).

The Act granted regulatory authority over nuclear energy to the Atomic Energy Commission. See Union of Concerned Scientists v. NRC, 735 F.2d 1437, 1443 n.1 (D.C. Cir. 1984). But the Energy Reorganization Act of 1974 disbanded that agency and redistributed its authority, as relevant here, to the Nuclear Regulatory Commission. Id. After Congress passed the Atomic Energy Act, commercial production of nuclear energy boomed.

Commercial nuclear energy is produced through a series of industrial processes, which include the mining and processing of nuclear fuel, the use of the fuel in a reactor, and the storage and ultimate disposal or reprocessing of that fuel. BRC Report at 9. Once nuclear fuel has been used in a reactor for about four to six years, it can no longer produce energy and is considered used or spent. *Id.* at 10. That spent fuel is removed from the reactor. *Id.*

Spent nuclear fuel is "fuel that has been withdrawn from a nuclear reactor following irradiation, the constituent elements of which have not been separated by reprocessing." 42 U.S.C. § 10101(23). It's "intensely radioactive" and "must be carefully stored." Pac. Gas & Elec. Co., 461 U.S. at 195. The spent fuel is first placed in wet pool storage for cooling, where it

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remains for at least five years, but may remain for decades. BRC REPORT at 11. Once the spent nuclear fuel has cooled sufficiently in wet storage, it's generally transferred to dry cask storage. *Id.*

At first, there was little concern regarding storage for spent fuel. See BRC Report at 19-20; Idaho v. DOE, 945 F.2d 295, 298-99 (9th Cir. 1991). There was a widespread belief within the commercial nuclear energy industry that spent fuel would be reprocessed. Idaho, 945 F.2d 295, 298-99 (9th Cir. 1991). But the private reprocessing industry collapsed in the 1970s, id., and growing concerns led President Ford to issue a directive deferring commercial reprocessing and recycling, which President Carter later extended. BRC Report at 20. Although President Reagan reversed that policy, "for a variety of reasons, including costs, commercial reprocessing has never resumed." Id.

After years of accumulating spent nuclear fuel in nuclear power plants throughout the country, see 42 U.S.C. § 10131(a)(3), Congress enacted the Nuclear Waste Policy Act in 1982. That Act sought to "devise a permanent solution to the problems of civilian radioactive waste disposal." Id. It tasked the Department of Energy with establishing "a repository deep underground within a rock formation where the waste would be placed, permanently stored, and isolated from human contact." Nat'l Ass' of Regul. Util. Comm'rs v. DOE, 680 F.3d 819, 821 (D.C. Cir. 2012). Yucca Mountain in Nevada was chosen as the only suitable site for the repository. See 42 U.S.C. § 10172. The decision drew widespread opposition in Nevada. BRC REPORT at 22.

Decades of delay ensued. Despite a Congressional mandate that the Department of Energy start accepting waste from the States by January 31, 1998, see 42 U.S.C. § 10222(a)(5)(B), "by the mid-1990s, the Department of Energy made clear that it could not meet the 1998 deadline, and it came and

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went without the federal government accepting any waste." Texas v. U.S., 891 F.3d 553, 555-56 (5th Cir. 2018).

In 2008, the Department of Energy finally submitted its license application for the Yucca Mountain repository to the Commission. *In re Aiken Cnty.*, 725 F.3d 742, 258 (D.C. Cir. 2013). But the Commission "shut down its review and consideration" of the application. *Id.* By its own admission, the Commission had no intention of reviewing the application, *id.*, even though the Nuclear Waste Policy Act mandates a decision be made within three years of submission. *See* 42 U.S.C. § 10134(d).

In light of the delays and controversy, the Obama Administration decided to halt the work on the Yucca Mountain repository. BRC REPORT at vi. The Obama Administration instead formed the Blue Ribbon Commission on America's Nuclear Future, which concluded that a consent-based approach to siting nuclear waste storage facilities would be preferred to the Yucca Mountain policy. See id. at vii–x.

Spent nuclear fuel continues to accumulate at reactor sites across the country. Some estimates suggest the U.S. inventory of spent nuclear fuel may exceed 200,000 metric tons by 2050. BRC REPORT at 14. The commercial nuclear power industry as a whole is estimated to generate between 2,000 and 2,400 metric tons of spent nuclear fuel each year. *Id.* And there are thousands of metric tons of spent fuel in various sites where commercial reactors no longer operate. *Id.*

B.

After the Blue Ribbon Commission embraced a consent-based approach for siting nuclear waste storage facilities, the governments of Texas and New Mexico expressed support for establishing facilities within the states. Then-Governors Rick Perry of Texas and Susana Martinez of New Mexico wrote letters supporting the establishment of facilities within their

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respective states. And Andrews County—a rural community located near the Texas-New Mexico border—passed a resolution in support of siting a spent nuclear fuel facility there.

Based in part on these expressions of support, Waste Control Specialists, LLC applied to the Commission for a license to operate a consolidated interim storage facility for high-level spent nuclear fuel in Andrews County. Andrews County is located within the Permian Basin, one of the country's largest oil basins and a top global oil producer.

The Commission began its environmental review of the proposed facility in accordance with the National Environmental Policy Act. See 42 U.S.C. § 4321 et seq. But the application anticipated that the Department of Energy would take title to the spent nuclear fuel. Some stakeholders challenged the legality of that provision as prohibited by the Nuclear Waste Policy Act. Waste Control Specialists then asked the Commission to suspend its review.

Approximately a year later, Interim Storage Partners, LLC—a partnership between the original applicant, Waste Control Specialists, and another company—asked the Commission to resume its review of the now-revised license application. In its summary report on the scoping period, the Commission noted that it had received comments expressing concerns that the facility would become a *de facto* permanent disposal facility and that the license would be illegal under existing regulations. The Commission responded that such comments were outside the scope of the environmental impact statement.

In December 2019, the Atomic Safety and Licensing Board—the independent adjudicatory division of the Commission—terminated an adjudicatory proceeding regarding the license application. Before the proceeding was terminated, Fasken timely filed five contentions alleging that the Case: 21-60743 Document: 00516873781 Page: 7 Date Filed: 08/25/2023

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Commission violated the National Environmental Policy Act and its own regulations. The Board denied each one. The following month, Fasken filed a motion to reopen the record along with a motion to amend a previously filed contention. The Board denied the motions.

The Commission published a draft environmental impact statement in May 2020. The Commission received approximately 2,527 unique comments on the draft environmental impact statement, and many opposed the facility. One comment was a letter from Texas Governor Greg Abbott urging the Commission to deny the license application because of the lack of a permanent repository and the importance of the Permian Basin to the nation's energy security and economy. The Texas Commission on Environmental Quality submitted a comment that the licensing lacks public consent and doesn't properly account for the possibility that Texas would become the permanent solution of spent nuclear fuel disposal if the permanent repository isn't developed by the expiration of the facility's 40-year license term.

Fasken also submitted various comments. Its comments noted the uniqueness of the Permian Basin, the danger of transporting spent nuclear fuel to the facility, the lack of community consent, and the possibility that the facility could become a *de facto* permanent facility. Based on the draft environmental impact statement, Fasken also filed a second motion to reopen the adjudicatory proceeding. The Board once again denied the request.

The Commission issued the final environmental impact statement in July 2021. It recommended the license be issued, and noted that concerns regarding Yucca Mountain and the need for a permanent repository fell outside its scope. In an appendix, the Commission responded to timely comments, including those from Petitioners. The Commission responded to concerns that the facility would become a *de facto* permanent repository by noting the application was only for a temporary facility.

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The following September, the Texas Legislature passed H.B. 7. The statute makes it illegal to "dispose of or store high level radioactive waste" in Texas. Governor Abbott sent a letter to the Commission with a copy of H.B. 7. He reiterated that "the State of Texas has serious concerns with the design of the proposed ISP facility and with locating it in an area that is essential to the country's energy security." The next day, Fasken submitted an environmental analysis critiquing various aspects of the final environmental impact statement.

A few days later, the Commission issued the license.

Texas and Fasken have now petitioned this court for review of the license. Texas asks that the license be set aside. And Fasken asks that we suspend all further activities on the facility and remand to the Commission for a hard look analysis. While this case was pending before this court, Fasken and others who sought but were denied intervention in the agency adjudication had a petition for review pending before the D.C. Circuit appealing the denials of their intervention. See Don't Waste Michigan v. NRC, 2023 WL 395030 (Jan. 25, 2023). The petition was denied in January 2023. Id. at *1. Interim Storage Partners, LLC intervened in this case to represent its interests.

II.

We begin with jurisdiction. The Commission challenges this court's jurisdiction to hear the petitions for review for lack of both constitutional standing and statutory standing. We consider each argument in turn and find neither succeeds.

A.

As a preliminary matter, the Commission suggests that Petitioners forfeited constitutional standing by failing to argue it in their opening briefs. We disagree.

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Neither Petitioner argued constitutional standing beyond their general jurisdictional statements. Generally, a petitioner is required "to present specific facts supporting standing through citations to the administrative record or affidavits or other evidence attached to its opening brief, unless standing is self-evident." Sierra Club v. EPA, 793 F.3d 656, 662 (6th Cir. 2015) (emphasis added, quotation omitted). A petitioner may reasonably believe standing to be self-evident when "nothing in the record alerted [the] petitioners to the possibility that their standing would be challenged." Am. Libr. Ass'n v. FCC, 401 F.3d 489, 492 (D.C. Cir. 2005). That's the case here.

From the earliest stages of this proceeding, the Commission has challenged jurisdiction on statutory standing grounds only. It twice moved to dismiss, but neither motion challenged constitutional standing. Accordingly, Petitioners could reasonably assume it was self-evident. *Cf. Ctr. for Biological Diversity v. EPA*, 937 F.3d 533, 542 n.4 (5th Cir. 2019) ("overlook[ing] Petitioners' decision to include only a cursory discussion of standing because . . . they had a good-faith (though mistaken) belief that standing would be both undisputed and easy to resolve"). And—once constitutional standing was challenged—both Petitioners provided well-developed legal arguments with citations to the record and evidence to show their standing. Petitioners haven't forfeited constitutional standing.

The "irreducible constitutional minimum" of standing requires that Petitioners "must have (1) suffered an injury in fact, (2) that is fairly traceable to the challenged conduct of the defendant, and (3) that is likely to be redressed by a favorable judicial decision." Spokeo, Inc. v. Robins, 578 U.S. 330, 338 (2016). The causation elements of the constitutional standing analysis are easily met: Petitioners' alleged injuries directly result from the issuance of the license (traceability), and an order from this court could vacate the license (redressability). So only injury in fact is at issue.

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The Commission argues that the licensing and eventual operation of the storage facility doesn't injure either Texas or Fasken. We disagree. Because "the presence of one party with standing is sufficient to satisfy Article III's case-or-controversy requirement," we may proceed even if only one of the Petitioners has standing. *Rumsfeld v. FAIR*, 547 U.S. 47, 52 n.2 (2006). But here both Petitioners successfully assert an injury resulting from the license.

Texas meets the injury-in-fact requirement because the license preempts state law. Texas has "a sovereign interest in the power to create and enforce a legal code." Tex. Off. of Pub. Util. Counsel v. FCC, 183 F.3d 393, 449 (5th Cir. 1999) (quotation omitted) (holding that Texas has standing to challenge the FCC's assertion of authority over an aspect of telecommunications regulation that the State believed it controlled). And we have held that the preemption of an existing state law can constitute an injury. Texas v. United States, 787 F.3d 733, 749 (5th Cir. 2015). "A state has standing based on a conflict between federal and state law if the state statute at issue regulates behavior or provides for the administration of a state program, but not if it simply purports to immunize state citizens from federal law." Id. (cleaned up). Here the issuance of the license and resulting operation of the facility directly conflicts with H.B. 7.

The Texas Legislature has enacted legislation that prevents the storage of high-level radioactive waste, including spent nuclear fuel, within the State except at currently or formerly operating nuclear power reactors. The legislation also amends Texas statutes to add that "a person, including the compact waste disposal facility license holder, may not dispose of or store high level radioactive waste in this state." Tex. Health & Safety Code § 401.072. Although a non-binding, declaratory state statute would not be enough to confer standing, here there's an enforceability conflict between the license and operation of the facility, which authorizes storage of

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high-level radioactive waste in Texas, and H.B. 7, which proscribes such storage. *Cf. Virginia v. Sebelius*, 656 F.3d 253, 270 (4th Cir. 2011) (a state statute that is merely a "non-binding declaration [and] does not create any genuine conflict . . . creates no sovereign interest capable of producing injury-infact"). That's enough for Texas to assert an injury.

Fasken also has standing based on its proximity to radioactive materials. To establish injury in an environmental case, there's a "geographicnexus requirement." Biological Diversity, 937 F.3d at 538. "The Supreme Court has ruled that geographic remoteness forecloses a finding of injury when no further facts have been brought forward showing that the impact in those distant places will in some fashion be reflected where the plaintiffs are." Id. (cleaned up). See also id. at 540 ("when a person visits an area for aesthetic purposes, pollution interfering with his aesthetic enjoyment may cause an injury in fact," if "the aesthetic experience was actually offensive to the plaintiff"). Fasken has provided evidence of its members' geographic proximity to the facility. Some of Fasken's members own land within four miles of the facility, draw water from wells beneath the facility, drive within a mile of the facility, use rail lines the facility would use, and travel on highways within a few hundred feet of the rail lines that transport spent nuclear fuel to the facility. In the context of radioactive materials, such proximity is sufficient to establish injury. See Duke Power Co. v. Caroline Env't Study Grp., Inc., 438 U.S. 59, 74 (1978) ("[T]he emission of non-natural radiation into appellees' environment would also seem a direct and present injury."). See also Nuclear Energy Inst., Inc. v. EPA, 373 F.3d 1251, 1266 (D.C. Cir. 2004) (finding a petitioner living 18 miles from Yucca Mountain had standing); Kelley v. Selin, 42 F.3d 1501, 1509 (6th Cir. 1995) (finding petitioners who "own[] land in close proximity to . . . the proposed site for spent fuel storage" had "alleged sufficient injury to establish standing").

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PBLRO also has associational standing. "Associational standing is a three-part test: (1) the association's members would independently meet the Article III standing requirements; (2) the interests the association seeks to protect are germane to the purpose of the organization; and (3) neither the claim asserted, nor the relief requested requires participation of individual members." Biological Diversity, 937 F.3d at 536 (quoting Texas Democratic Party v. Benkiser, 459 F.3d 582, 587 (5th Cir. 2006)). Each of those elements is met. First, some of its members have an injury because they live, work, or regularly drive close the facility. And as we've already noted, see supra, the causation elements are met. Next, "the germaneness requirement is undemanding and requires mere pertinence between the litigation at issue and the organization's purpose." Ass'n of Am. Physicians & Surgeons, Inc. v. Texas Med. Bd., 627 F.3d 547, 550 n.2 (5th Cir. 2010) (quotations omitted). This factor is easily met because PBLRO was created specifically to oppose the facility. Last, there's no reason to believe that PBLRO is unable to represent its members' interests without their individual participation. See id. at 551-53 (noting this prong usually isn't met when the relief sought is damages for individual members or the claim requires fact-intensive-individual inquiry).

R

Petitioners seeking to challenge a final order from the Commission also need standing under the Administrative Orders Review Act, generally known as the Hobbs Act. See Reytblatt v. NRC, 105 F.3d 715, 720 (D.C. Cir. 1997) ("[T]he Hobbs Act requires (1) 'party' status (i.e., that petitioners participated in the proceeding before the agency), and (2) aggrievement (i.e., that they meet the requirements of constitutional and prudential standing).") (citation omitted).

The Hobbs Act vests "exclusive jurisdiction to enjoin, set aside, suspend (in whole or in part), or determine the validity of . . . final orders of the"

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Commission on the federal courts of appeals. 28 U.S.C. § 2342. (The Act actually refers to the Atomic Energy Commission. But the Energy Reorganization Act of 1974 abolished that agency and transferred its licensing and related regulatory functions to the Nuclear Regulatory Commission. See 42 U.S.C. § 5841(a), (f).)

Under the Act, "[a]ny party aggrieved by the final order may . . . file a petition to review the order in the court of appeals wherein venue lies." 28 U.S.C. § 2344. Courts "have consistently held that the phrase 'party aggrieved' requires that petitioners have been parties to the underlying agency proceedings, not simply parties to the present suit." ACA Int'l v. FCC, 885 F.3d 687, 711 (D.C. Cir. 2018). See also Am. Trucking Ass'ns v. ICC, 673 F.2d 82, 84 (5th Cir. 1982) (per curiam) ("The word 'party' is used in a definite sense in the [Hobbs Act], and limits the right to appeal to those who actually participated in the agency proceeding."). The Commission argues that neither Texas nor Fasken has standing under the Hobbs Act because neither is a "party aggrieved."

"To be an aggrieved party, one must have participated in the agency proceeding under review." Wales Transp., Inc. v. ICC, 728 F.2d 774, 776 n.1 (5th Cir. 1984). Here, both Petitioners participated in the agency proceeding—Texas commented on its opposition of the issuance of the license and Fasken attempted to intervene and filed contentions. But according to the Commission, neither form of participation is sufficient to confer party status under the Hobbs Act.

The Commission argues that Texas doesn't have party status because "participating in the appropriate and available administrative procedures is the statutorily prescribed prerequisite to invocation of the Court's

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jurisdiction," and submitting comments doesn't accord with the degree of formality of the proceedings in this license adjudication.²

The Commission takes a different approach with Fasken. It argues that, as a party denied intervention, Fasken may only challenge the order denying it intervention. From the Commission's perspective, if a putative intervenor has failed to obtain party status, it can't later seek review of the final judgment on the merits.

The plain text of the Hobbs Act merely requires that a petitioner seeking review of an agency action be a "party aggrieved." 28 U.S.C. § 2344. The

It's also worth noting that caselaw suggests that so long as the petitioner is a "party aggrieved" and the basis for the challenge was brought before the agency by some party—even if not the by the petitioner—that's enough for the case to move forward. See Reytblatt, 105 F.3d at 720-21; Cellnet Commc'n, Inc. v. FCC, 965 F.2d 1106, 1109 (D.C. Cir. 1992). It'd make little sense to interpret the Hobbs Act as imposing an exhaustion requirement while allowing a petitioner to bring a claim it did not itself bring before the agency.

² In the alternative, the Commission argues that "even if this Court were to determine that dismissal of [Texas's] Petition for Review is not required as a matter of jurisdiction, the same result is nonetheless required as a matter of non-jurisdictional, mandatory exhaustion." Not so. The Commission relies on Fleming v. USDA, which held that "even nonjurisdictional exhaustion requirements . . . forbid judges from excusing nonexhaustion" and that "if the government raises [such an] exhaustion requirement, the court must enforce it." 987 F.3d 1093, 1099 (D.C. Cir. 2021). But neither the Hobbs Act nor the Atomic Energy Act impose a mandatory exhaustion requirement. Commission's argument implicitly equates the exhaustion requirements in the Horse Protection Act and the Prison Litigation Reform Act-both of which are discussed in Fleming—to the Hobbs Act and Atomic Energy Act. These statutes aren't comparable. Both the Horse Protection Act and the Prison Litigation Reform Act have explicit exhaustion requirements. See 7 U.S.C. § 6912(e) ("[A] person shall exhaust all administrative appeal procedures established by the Secretary [of Agriculture] or required by law before the person may bring an action in a court of competent jurisdiction."); 42 U.S.C. § 1997e(a) ("No action shall be brought with respect to prison conditions under section 1983 of this title, or any other Federal law, by a prisoner confined in any jail, prison, or other correctional facility until such an administrative remedies as are available are exhausted."). But neither the Hobbs Act nor the Atomic Energy Act do. See 28 U.S.C. § 2344 (no exhaustion requirement); 42 U.S.C. § 2239(b) (same).

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text makes no distinction between different kinds of agency proceedings. See Gage v. AEC, 479 F.2d 1214, 1218 (D.C. Cir. 1973). Nor does it suggest that a petitioner who went through the procedures to intervene in an adjudication can't be a party aggrieved. In fact, it's clear that the function of the "party aggrieved" status requirement is to ensure that the agency had the opportunity to consider the issue that petitioners are concerned with. See, e.g., id. at 1219 ("The 'party' status requirement operates to preclude direct appellate court review without a record which at least resulted from the fact-finder's focus on the alternative regulatory provisions which petitioners propose.") (emphases omitted).

In sum, the plain text of the Hobbs Act requires only that a petitioner have participated—in some way—in the agency proceedings, which Texas did through comments and Fasken did by seeking intervention and filing contentions. But caselaw suggests that's not enough.

Precedent from other circuits suggests that neither Texas nor Fasken are parties aggrieved for Hobbs Act purposes. The D.C. Circuit has read the Hobbs Act to contemplate participation in "the appropriate and available administrative procedures." *Id.* at 1217. And it has interpreted this to mean that the "degree of participation necessary to achieve party status varies according to the formality with which the proceeding was conducted." *Water Transp. Ass* 'n v. ICC, 819 F.2d 1189, 1192 (D.C. Cir. 1987). But see ACA Int'l, 885 F.3d at 711–712 (noting that in at least some limited circumstances commenting may be enough in certain non-rulemaking proceedings). The D.C. Circuit and at least one other circuit apply this heightened participation requirement. See Ohio Nuclear-Free Network v. NRC, 53 F.4th 236, 239 (D.C. Cir. 2022); Alabama Power Co. v. ICC, 852 F.2d 1361, 1368 (D.C. Cir. 1988). See also State ex rel. Balderas v. NRC, 59 F.4th 1112, 1117 (10th Cir. 2023). The D.C. Circuit has also said that, when an agency requires intervention, those who sought but were denied intervention lack standing to seek judicial

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review. Water Transp. Ass'n, 819 F.2d at 1192. See also NRDC v. NRC, 823 F.3d 641, 643 (D.C. Cir. 2016) ("To challenge the Commission's grant of a license renewal... a party must have successfully intervened in the proceeding by submitting adequate contentions under [the Commission's regulations].").

The D.C. Circuit embraces readings of the Hobbs Act that impose an extra-textual gloss by requiring a degree of participation not contemplated in the plain text of the statute. We think the fairest reading of the Hobbs Act doesn't impose such additional requirements. But we ultimately don't need to resolve that tension, because the Fifth Circuit recognizes an exception to the Hobbs Act party-aggrieved status requirement that's dispositive of this issue here.

This circuit recognizes an *ultra vires* exception to the party-aggrieved status requirement. In *American Trucking Associations, Inc. v. ICC*, this court noted "two rare instances" where a "person may appeal an agency action even if not a party to the original agency proceeding"—(1) where "the agency action is attacked as exceeding [its] power" and (2) where the person "challenges the constitutionality of the statute conferring authority on the agency." 673 F.2d at 85 n.4 (quotation omitted).³

³ The Commission's various arguments that this exception isn't applicable are unavailing. It's true that we've recognized the exception is "exceedingly narrow." Merchants Fast Motor Lines, Inc. v. ICC, 5 F.3d 911, 922 (5th Cir. 1993). And it's also true that other circuits have refused to adopt it. See Balderas, 59 F.4th at 1123-24; Nat'l Ass'n of State Util. Consumer Advocs. v. FCC, 457 F.3d 1238, 1250 (11th Cir. 2006); Erie-Niagara Rail Steering Comm. v. STB, 167 F.3d 111, 112-13 (2d Cir. 1999); In re Chicago, Milwakee, St. Paul & Pac. R.R., 799 F.2d 317, 334-35 (7th Cir. 1986). But the exception remains good law in this circuit. Neither the Commission nor the court have identified any case overturning the exception. And to the extent that the Commission claims the exception was mere dicta in American Trucking, that argument fails because we've since applied the

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This exception only allows us to reach those portions of the Petitioners' challenges that argue the Commission acted beyond its statutory authority. See Wales Transp., 728 F.2d at 776 n.1 (allowing petitioner to proceed despite not having participated in the agency proceeding on only those claims that challenged the agency's authority under the statute). Accordingly, we must consider which, if any, of the Petitioners' challenges fall within that category.

Texas makes three merits arguments: (1) the Commission lacks the statutory authority to license the facility; (2) the license issuance violated the Administrative Procedure Act; and (3) the Commission violated the National Environmental Policy Act by failing to assess the risks of a potential terrorist attack. The first argument falls within the exception. It attacks the Commission for licensing a facility without the authority to do so under the Atomic Energy Act, and in conflict with the Nuclear Waste Policy Act.

Fasken makes four merits arguments: (1) the Commission violated the National Environmental Policy Act and Administrative Procedure Act by allowing a licensing condition that violates the Nuclear Waste Policy Act; (2) the Commission's assumptions about when the permanent repository will be operational are arbitrary and capricious; (3) the Commission adopted an unreasonably narrow purpose statement; and (4) the Commission violated the National Environmental Policy Act and Administrative Procedure Act by

exception in Wales Transportation, Inc. v. ICC, 728 F.2d 774, 776 n.1 (5th Cir. 1984). Under our circuit's rule of orderliness, we are bound to follow American Trucking and Wales Transportation because they haven't been overturned by the en banc court. The Commission is also wrong in suggesting the exception is limited to challenges of ICC orders. While it's true that both American Trucking and Wales Transportation involved challenges to ICC orders, neither case limits the exception's application to the ICC. See Am. Trucking, 673 F.2d at 85 n.4 (referring to agency proceedings, not ICC proceedings); Wales Transp., 728 F.2d at 776 n.1 (same).

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accepting the applicant's unreasonable site selection. The first of these challenges falls within the exception. Fasken's argument centers on the contention that the Commission acted beyond its statutory authority by issuing a license with a condition expressly prohibited by the Nuclear Waste Policy Act.

III.

The Commission has no statutory authority to issue the license. The Atomic Energy Act doesn't authorize the Commission to license a private, away-from-reactor storage facility for spent nuclear fuel. And issuing such a license contradicts Congressional policy expressed in the Nuclear Waste Policy Act. This understanding aligns with the historical context surrounding the development of these statutes.

A.

Under the Atomic Energy Act, the Commission retains jurisdiction over nuclear plant licensing and regulation. See 42 U.S.C. § 5842. It has authority to regulate the construction and operation of nuclear power plants. See 42 U.S.C. §§ 2011–2297h-13. See also Union of Concerned Scientists, 735 F.2d at 1438–39 (summarizing the two-step licensing procedure for nuclear power plant operation).

The Act also confers on the Commission the authority to issue licenses for the possession of "special nuclear material," see 42 U.S.C. § 2073, "source material," see id. § 2093, and "byproduct material," see id. § 2111. See also 42 U.S.C. §§ 2014(aa), (z), (e) (defining each term, respectively). Special nuclear material, source material, and byproduct material are constituent materials of spent nuclear fuel. See Bullcreek v. NRC, 359 F.3d 536, 538 (D.C. Cir. 2004). The Commission argues that, because it has authority to issue licenses for the possession of these constituent materials, that means it has broad authority to license storage facilities for spent nuclear fuel.

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But this ignores the fact that the Act authorizes the Commission to issue such licenses only for certain enumerated purposes—none of which encompass storage or disposal of material as radioactive as spent nuclear fuel.

Sections 2073 and 2093 specify that licenses may be issued for various types of research and development, see 42 U.S.C. §§ 2073(a)(1)-(a)(2), 2093(a)(1)-(a)(2). It also permits such other uses that the Commission either "determines to be appropriate to carry out the purposes of th[e] chapter," id. § 2073(a)(4), or "approves . . . as an aid to science and industry," id. § 2093(a)(4). Principles of statutory interpretation require these grants be read in light of the other, more specific purposes listed—namely for certain types of research and development. Cf. U.S. v. Jicarilla Apache Nation, 564 U.S. 162, 185 (2011) ("When Congress provides specific statutory obligations, we will not read a 'catchall' provision to impose general obligations that would include those specifically enumerated.").

Both these sections also allow the agency to issue licenses "for use under a license issued pursuant to section 2133 of th[e] title." *Id.* 42 U.S.C. §§ 2073(a)(3), 2093(a)(3) (same). Section 2133 details the Commission's authority to issue licenses for "utilization or production facilities for industrial or commercial purposes." *Id.* § 2133(a). Utilization and production have specific definitions under the statute. *See id.* §§ 2014 (cc) (defining utilization facilities); 2014(v) (defining production facilities). And the definitions of utilization and production facilities are about nuclear reactors and fuel fabrication or enrichment facilities—not storage or disposal, as the Commission admits in its briefing. *See id.* Neither § 2073 nor § 2093 confers a broad grant of authority to issue licenses for any type of possession of special nuclear material or source material.

The same is true for § 2111. That section authorizes the Commission "to issue general or specific licenses to applicants seeking to use byproduct

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material for research or development purposes, for medical therapy, industrial uses, agricultural uses, or such other useful applications as may be developed." *Id.* § 2111(a). It also specifies conditions under which certain types of byproduct material may be disposed. *Id.* § 2111(b). And the types of byproduct material covered by § 2111(b) emit radiation for significantly less time than spent nuclear fuel.

That section cross-references the definition of byproduct materials in § 2014(e)(3)-(4), which refers to radium-226 and other material that "would pose a threat similar to the threat posed by . . . radium-226 to the public health and safety." That's important because some of the isotopes in spent nuclear fuel have much longer half-lives than radium-226. The "intensity of radiation from radioactive materials decreases over time" and the "time required for the intensity to decrease by one-half is referred to as the 'half-life.'" NRC, FREQUENTLY ASKED QUESTIONS (FAQS) REGARDING RAhttps://scp.nrc.gov/narmtoolbox/ra-8 A.1, **DIUM-226** dium%20faq102008.pdf. Radium-226 has a half-life of 1600 years. Id. Spent nuclear fuel, on the other hand, is composed of a variety of radioactive isotopes of elements produced in the nuclear fission process. NRC, RADIO-BACKGROUNDER ACTIVE WASTE https://www.nrc.gov/docs/ML0501/ML050110277.pdf. Some of these isotopes - strontium-90 and cesium-137 - have half-lives of about 30 years. But others "take much longer to decay." Id. One of these isotopes is plutonium-239, which "has a half-life of 24,000 years"-fifteen times that of radium-226. Id. There's no plausible argument that spent nuclear fuel, which contains radioactive isotopes with half-lives much longer than radium-226, is the type radioactive material contemplated in the disposal provision in § 2111(b).

So these provisions do not support the Commission's claim of authority. In response, the Commission and Interim Storage Partners, LLC point to two cases from sister circuits. Both are unpersuasive.

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In Bullcreek v. NRC, the D.C. Circuit denied petitions for review of the Commission's Rulemaking Order and held that the Nuclear Waste Policy Act did "not repeal or supersede the [Commission]'s authority under the Atomic Energy Act to license private away-from-reactor storage facilities." 359 F.3d at 537-38. The D.C. Circuit essentially assumed that the Atomic Energy Act had granted the Commission authority to license away-from-reactor storage facilities, despite explicitly recognizing that the Act "does not specifically refer to the storage or disposal of spent nuclear fuel." Id. at 538. Rather than focus on the text of the statute, it merely noted that "it has long been recognized that the [Atomic Energy Act] confers on the [Commission] authority to license and regulate the storage and disposal of such fuel." Id. But none of the cases the D.C. Circuit cited provide a textual analysis of the Atomic Energy Act and whether it allows away-from-reactor spent nuclear fuel storage. Each of those cases dealt with separate questions of preemption and the role of states in this scheme. See generally Pac. Gas. & Elec. v. State Energy Res. Conservation & Dev. Comm'n, 461 U.S. 190 (1983); Jersey Cent. Power & Light Co. v. Twp. of Lacey, 772 F.2d 1103 (3d Cir. 1985); Illinois v. Gen. Elec. Co., 683 F.2d 206 (7th Cir. 1982). They are irrelevant to the question before us.

So the D.C. Circuit provided no textual basis for its assumption that the statute authorized the Commission to issue such licenses. See id. (discussing the Atomic Energy Act). Bullcreek may be correct that the Nuclear Waste Policy Act didn't repeal portions of the Atomic Energy Act since "repeals by implication are not favored," but it doesn't actually address what authority the Commission had under the Atomic Energy Act. Morton v. Mancari, 417 U.S. 535, 549 (1974).

The other case the Commission cites—Skull Valley Band of Goshute Indians v. Nielson, 376 F.3d 1223 (10th Cir. 2004)—is just as unhelpful. It merely relies on Bullcreek to "not revisit the issues surrounding the

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[Commission]'s authority to license away-from-reactor [spent nuclear fuel] storage facilities." *Skull Valley*, 376 F.3d at 1232. It too assumes the Commission's authority without analyzing the statute.

B.

Moreover, the Commission's argument cannot be reconciled with the Nuclear Waste Policy Act.

Spent nuclear fuel wasn't a concern in the 1940s and 1950s when the Atomic Energy Act was passed and amended. "Prior to the late 1970's, private utilities operating nuclear reactors were largely unconcerned with the storage of spent nuclear fuel." *Idaho*, 945 F.2d at 298. "It was accepted that spent fuel would be reprocessed." *Id.* "In the mid-70's, however, the private reprocessing industry collapsed for both economic and regulatory reasons." *Id.* "As a consequence, the nuclear industry was confronted with an unanticipated accumulation of spent nuclear fuel, inadequate private facilities for the storage of the spent fuel, and no long term plans for managing nuclear waste." *Id. See also* BRC Report at 20 (noting these problems and describing passage of the Act as "mark[ing] the beginning of a new chapter in U.S. efforts to deal with the nuclear waste issue"). This led Congress to pass the Nuclear Waste Policy Act in 1982.

The Nuclear Waste Policy Act provides a comprehensive scheme to address the accumulation of nuclear waste. Congress recognized that "Federal efforts during the [prior] 30 years to devise a permanent solution to the problems of civilian radioactive waste disposal ha[d] not been adequate" and that "State and public participation in the planning and development of repositories is essential in order to promote public confidence in the safety of disposal of such waste and spent fuel." 42 U.S.C. § 10131(a)(3), (6). "The Act made the federal government responsible for permanently disposing of spent nuclear fuel and high-level radioactive waste produced by civilian

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nuclear power generation and defense activities." Nat'l Ass'n of Regul. Util. Comm'rs v. DOE, 680 F.3d 819, 821 (D.C. Cir. 2012). See also 42 U.S.C. § 10131(a)(4) ("[T]he Federal Government has the responsibility to provide for the permanent disposal of high-level radioactive waste and such spent nuclear fuel as may be disposed of in order to protect the public health and safety and the environment.").

The Act also tasked the Department of Energy with establishing "a repository deep underground within a rock formation where the waste would be placed, permanently stored, and isolated from human contact." Nat'l Ass'n of Regul. Util Comm'rs, 680 F.3d at 821. See also 42 U.S.C. §§ 100133–34 (tasking the Energy Secretary with site characterization and public hearing duties related to the Yucca Mountain site selection). Yucca Mountain was chosen as the only suitable site for the repository when the Act was amended in 1987. See 42 U.S.C. § 10172 (selection of Yucca Mountain site). But the project stalled, even though the Nuclear Waste Policy Act "is obviously designed to prevent the Department [of Energy] from delaying the construction of Yucca Mountain as the permanent facility while using temporary facilities." Nat'l Ass'n of Regul. Util. Comm'rs v. DOE, 736 F.3d 517, 519 (D.C. Cir. 2013) (citing 42 U.S.C. § 10168(d)(1)).

In addition to the establishment of the permanent repository, see 42 U.S.C. §§ 10131-10145, the Nuclear Waste Policy Act also established other measures to deal with spent nuclear fuel.⁴

⁴ All these measures are subject to the proviso in 42 U.S.C. § 10155(h), which states that "nothing in this chapter shall be construed to encourage, authorize, or require the private or Federal use, purchase, lease, or other acquisition of any storage facility located away from the site of any civilian nuclear power reactor and not owned by the Federal Government on" the date of enactment.

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One is temporary storage. See id. §§ 10151-10157. The Act places "primary responsibility for providing interim storage of spent nuclear fuel" on "the persons owning and operating civilian nuclear power reactors." Id. § 10151(a)(1). It tasks the Commission and the Secretary of Energy to "take such actions as . . . necessary to encourage and expedite the effective use of available storage, and the necessary additional storage, at the site of each civilian nuclear power reactor." Id. § 10152 (emphasis added). See also id. § 10153 ("The establishment of such procedures shall not preclude the licensing . . . of any technology for the storage of civilian spent nuclear fuel at the site of any civilian nuclear power reactor.") (emphasis added). It further tasks the Secretary of Energy with "provid[ing] . . . capacity for the storage of spent nuclear fuel from civilian nuclear power reactors." Id. § 10155(a)(1). Moreover, the Act provides that "the Federal Government has the responsibility to provide . . . not more than 1,900 metric tons of capacity for interim storage of spent nuclear fuel for civilian nuclear power reactors that cannot reasonably provide adequate storage capacity" where it is necessary for the "continued, orderly operation of such reactors." Id. § 10151(a)(3). Moreover, the Act provides that "the Federal Government has the responsibility to provide . . . not more than 1,900 metric tons of capacity for interim storage of spent nuclear fuel for civilian nuclear power reactors that cannot reasonably provide adequate storage capacity" where it is necessary for the "continued, orderly operation of such reactors." Id. § 10151(a)(3). Here, the license permits storage of at least 5,000 and as much as 40,000 metric tons of nuclear waste.

The other measure is monitored retrievable storage. See id. § 10161-10169. See also id. § 10101(34) (defining "monitored retrievable storage facility"). Under the statute, "[t]he Secretary [of Energy] is authorized to site, construct, and operate one monitored retrievable storage facility subject to the conditions described [in the relevant sections of statute]." Id. § 10162(b).

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And one of those conditions is that "[a]ny license issued by the Commission for a monitored retrievable storage facility under [the statute] shall provide that . . . construction of such facility may not begin until the Commission has issued a license for the construction of a repository [i.e., Yucca Mountain]." *Id.* § 10168(d)(1).

Reading these provisions together makes clear that the Nuclear Waste Policy Act creates a comprehensive statutory scheme for addressing spent nuclear fuel accumulation. The scheme prioritizes construction of the permanent repository and limits temporary storage to private at-the-reactor storage or at federal sites. It plainly contemplates that, until there's a permanent repository, spent nuclear fuel is to be stored onsite at-the-reactor or in a federal facility.

In sum, the Atomic Energy Act doesn't authorize the Commission to license a private, away-from-reactor storage facility for spent nuclear fuel. And the Nuclear Waste Policy Act doesn't permit it. Accordingly, we hold that the Commission doesn't have authority to issue the license challenged here.

When read alongside each other, we find these statutes unambiguous. And even if the statutes were ambiguous, the Commission's interpretation wouldn't be entitled to deference.

Last year, the Supreme Court directed that, "[w]here the statute at issue is one that confers authority upon an administrative agency, that inquiry must be shaped, at least in some measure, by the nature of the question presented—whether Congress in fact meant to confer the power the agency has asserted" and whether there are "reason[s] to hesitate before concluding that Congress meant to confer such authority." West Virginia v. EPA, 142 S. Ct. 2587, 2607-08 (2022) (quotations omitted) (adopting the major questions doctrine).

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Disposal of nuclear waste is an issue of great "economic and political significance." *Id.* at 2608. What to do with the nation's ever-growing accumulation of nuclear waste is a major question that—as the history of the Yucca Mountain repository shows—has been hotly politically contested for over a half century. Congress itself has acknowledged that "high-level radioactive waste and spent nuclear fuel have become major subjects of public concern." 42 U.S.C. § 10131(a)(7) (findings section of the Nuclear Waste Policy Act). "A decision of such magnitude and consequence rests with Congress itself, or an agency acting pursuant to *clear* delegation from that representative body." *West Virginia*, 142 S. Ct. at 2616 (emphasis added). Here, there's no such clear delegation under the Atomic Energy Act. And the Nuclear Waste Policy Act belies the Commission's arguments to the contrary.

* * *

We grant the petitions for review, vacate the license, and deny the Commission's motions to dismiss.

Sup 5 for CON-Casks

Feb 26. 2024 - RW City Council

Xcel Energy*

2024-2040 Upper Midwest Integrated Resource Plan (IRP)

Docket No. E002/RP-24-67

February 2024

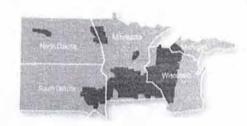
City Council

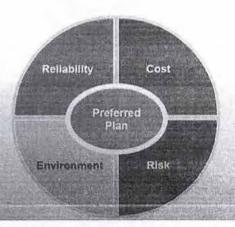
February 26, 2024

Upper Midwest IRP Overview

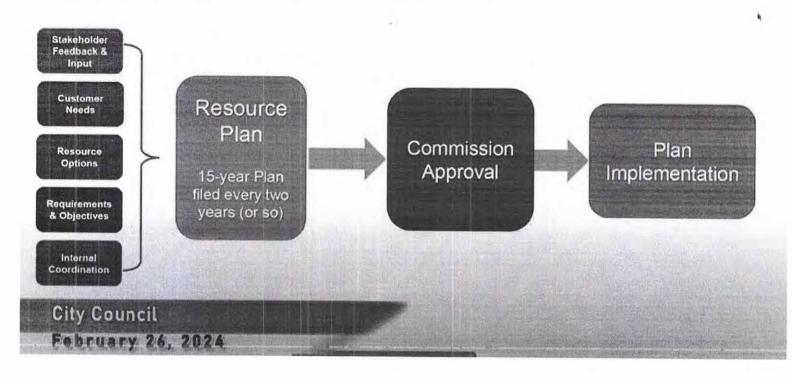
- 15-year plan for Upper Midwest service area
- Capacity expansion modeling to assess size, type, timing of least-cost resource sets that can serve future load
- Balances four core planning objectives

City Council February 26, 2026.



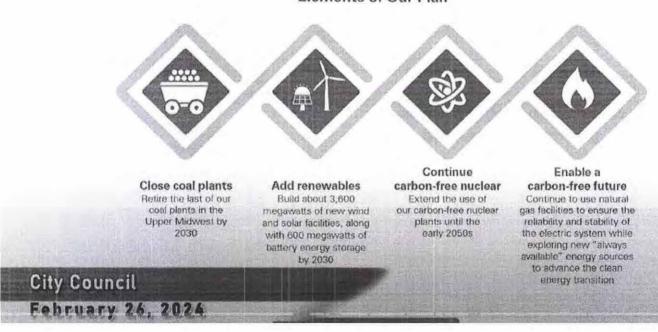


Integrated Resource Planning Process



Upper Midwest Energy Plan

Elements of Our Plan

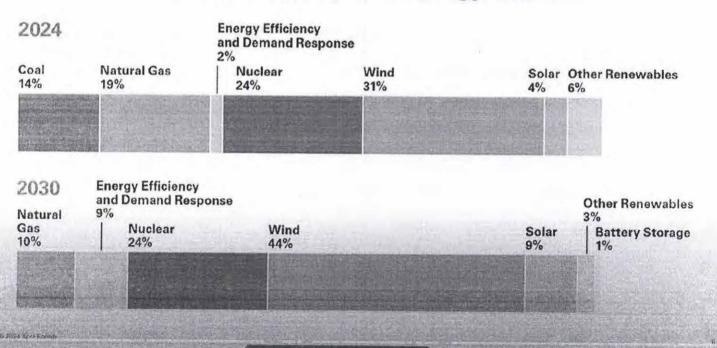


Preferred Plan Overview

- Extension of nuclear facilities: Prairie Island (20 years) and Monticello (10 years)
- Plans to extend MN Refuse-Derived Fuel (RDF) Waste-to-Energy plants (10 years)
- · Additional wind, solar, and battery storage resources added
- Utilize "always available" resources, including natural gas, to back up wind and solar
- · Coal plant retirements on track
- Preferred plan charts a path toward achieving Minnesota's new "100 by 2040" law
- · Load growth expected: data centers, electric vehicles, and beneficial electrification
- Maintains critical reliability and affordability for our customers at <1% average
 City Council
 the national average

February 26, 2024

Our Customers' Energy Mix



Public Meetings

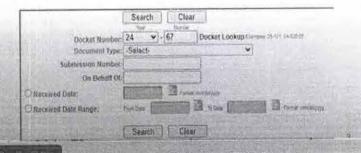
- · Multiple Stakeholder engagement workshops held
- · PUC is coordinating future public meetings
- Designed for the public to learn about the process, hear about the plan, and provide input
- Expected early to mid-June; dates & locations TBD; virtual option available
- · Red Wing/Prairie Island area has been identified as a location

Integrated Resource Plan - What's Next?

- 1. Interested parties can file for intervenor status and ask discovery questions
- 2. Comment period closes: June 28, 2024
 - Reply comment period closes: August 28, 2024
 - Supplemental comment period closes: September 30, 2024
- Public meetings: expected early to mid-June 2024 (determined by ALJ)
 - Advertised social media, bill inserts, newspapers ads, etc.
- 4. PUC hearing by February 1, 2025
 - Docket no. 24 (year) 67 (number)

to arrive some thinking

MN DOC E-filing (state.mn.us)







CERTIFICATE OF NEED (CON) – DRY FUEL STORAGE

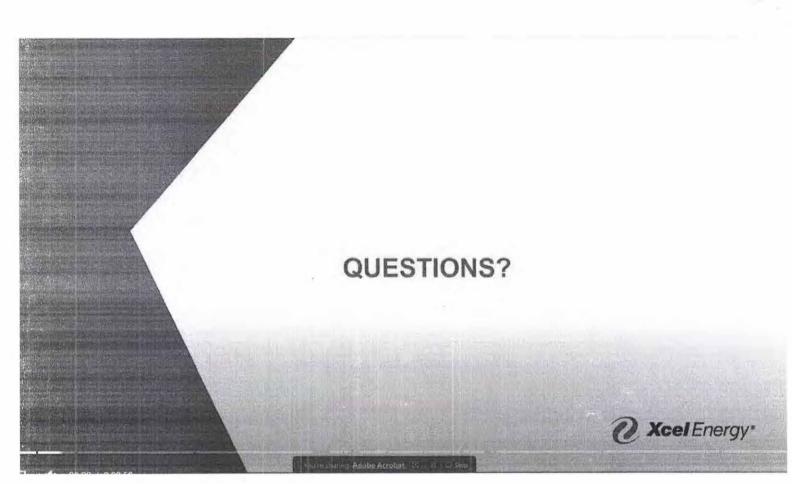
Docket No. E002/CN-24-68

February 2024

Certificate of Need (CON) - What's Next?

- 1. Open comment period for determination of application completeness
 - Closes March 8, 2024
 - Company responses due March 15, 2024
- 2. Department of Commerce Energy Environmental Review and Analysis develops Environmental Impact Statement (EIS) – TBD
- 3. Public meetings and comment periods associated with the EIS draft
- 4. Administrative Law Judge (ALJ) will be assigned to case
- Testimony and public hearings
- 6. ALJ issue final report to PUC
- 7. PUC decision
- Legislative stay





BUSINESS

The real cause of Xcel's lengthy Prairie Island nuclear outage: Workers drilled through cables

The company said it did not use ground radar that would have shown where the underground cables were in the Red Wing plant and admitted its excavation planning and oversight was inadequate.

By Walker Orenstein (https://www.startribune.com/walker-orenstein/11331478/) Star Tribune

APRIL 12, 2024 — 8:02AM

Xcel Energy initially pinpointed the cause of a lengthy outage at its Prairie Island nuclear power plant <u>as an equipment issue (https://www.startribune.com/xcels-prairie-island-nuclear-plant-will-be-out-of-commission-until-january/600321701/?refresh=true)</u> between the turbine and the electric grid.

But that didn't quite tell the full story: Xcel workers at the plant actually cut a bundle of power cables when drilling sideways underground in October, interrupting power to some of the Red Wing plant's equipment and causing one of the two reactors to shut down.

Xcel told federal nuclear regulators last month it did not use ground radar in an area that would have shown the cables' location. The company also said its excavation planning and oversight was inadequate, admitting to "procedural weaknesses and poor communications" between departments.

"This resulted in work progressing in the field without all controls in place that would be expected for work at a nuclear plant," an Xcel filing with the U.S. Nuclear Regulatory Commission read.



Dave Lochbaum, a nuclear safety engineer formerly with the Union of Concerned Scientists, said the plant's systems worked as intended when the reactor safely shut down. But he said cutting into power cables could have been a safety risk for the workers, which Xcel acknowledged as well.

Prairie Isla 2617

"We know that any electrical contact can potentially be hazardous," Xcel spokesman Kevin Coss said. "None of the workers operating the drilling equipment were hurt when it impacted the cables. To prevent similar issues in the future, we have made changes to how we approach excavation projects as well as our oversight of workers coming to the plant for projects."

In April, Xcel told the Minnesota Department of Commerce it realized the damaged cables were aging and at risk of water damage after inspecting them. That would have likely caused both units at Prairie Island to shut down in the future, Xcel said.

ADVERTISEMENT

In March, Commerce also asked why Xcel customers should pay for the Unit 1 outage. In regulatory filings, the company said customers should cover the cost of fuel and power it bought on the energy market in place of nuclear energy during the Unit 1 outage because, despite causing the problem, Xcel was operating prudently.

"Prudence does not mean perfection, and even prudent decisions and operations can sometimes result in undesirable outcomes," Xcel wrote.

Unit 1 at Prairie Island went down on Oct. 19. That caused a longer-than-expected outage at the plant's second unit, too, which had powered down two weeks earlier for scheduled refueling and maintenance.

Xcel expected both reactors to be up and running in January, but the company <u>did not begin running both units at the plant again consistently until mid March (https://www.startribune.com/xcel-energy-prairie-island-nuclear-power-plant-puc-utility-commerce/600352107/)</u>.

The Xcel workers drilling underground were helping to replace a different power cable between an electric substation and the plant. The company replaced the damaged cables

rather than repair them. Xcel also took time when the plant was offline to do a more thorough plant inspection than it could have otherwise, among other maintenance, which will avoid the need to shut down both units in the future for that work.

The aging nuclear plants are a crucial part of Xcel's plans for a carbon-free energy grid by 2040 (https://www.startribune.com/xcel-aging-nuclear-plants-risks-monticello-prairie-island-red-wing-minnesota-clean-energy-2040-plan/600282822/). Nuclear makes up about 30% of the utility's power supply in the Upper Midwest.

Xcel wants to extend the life of its two units at Prairie Island (https://www.startribune.com/xcel-energy-long-term-plan-prairie-island-nuclear-gas-plants-wind-solar-large-scale-battery/600340390/) so they can run until 2053 and 2054, 20 years beyond when their current licenses expire, something that requires state and federal approval. Xcel has also asked the Public Utilities Commission for permission to store more nuclear waste at the site (https://www.startribune.com/xcel-prairie-island-plant-more-nuclear-storage/600342090/) in above-ground casks.

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